



ALPHA DECAY  
OF EXCITED STATES OF CARBON-12

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## ABSTRACT

A crystal of the carbon-rich phosphor, anthracene, has been bombarded by monoenergetic fast neutrons. Alpha-particle decays of carbon nuclei have been separated from other events by a specialised application of the pulse shape discrimination technique. A numerical analysis of the data has yielded alpha-particle energy spectra over a wide energy range.

A counter has been assembled to detect neutrons scattered by carbon nuclei in the crystal, and the measurements repeated in coincidence with these neutrons. The energy range of particle resolving power has been extended. Structure in the resulting alpha-particle and proton energy spectra is attributed to particle decays of specific levels in carbon-12. It is shown how the experiment may be modified and extended to measure decay parameters in absolute terms.

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## TABLE OF CONTENTS

CHAPTER 1	INTRODUCTION	
1.1	Alpha Decay Studies	1
1.2	Organic Phosphors	3
1.3	Objects of the Present Investigation	7
CHAPTER 2	EXPERIMENTAL ARRANGEMENTS	
2.1	Neutron Production	8
2.2	Neutron Detection	10
2.3	Pulse Shape Discrimination System	11
2.4	Coincidence System	13
2.4A	Calibration Procedure	15
2.5	Neutron Time of Flight System	19
2.5A	Calibration Procedure	20
CHAPTER 3	MEASUREMENT OF ALPHA SPECTRA	
3.1	The Dual Parameter Experiment	25
3.2	Reduction of Data to Energy Spectra	27
3.3	Discussion of Results	30
3.4	The Coincidence Experiment	37
CHAPTER 4	MEASUREMENT OF NEUTRON SCATTERING CROSS SECTIONS	
4.1	The Neutron Time of Flight Experiment	45
4.2	Discussion of Results	47
4.3	Search for an Effective Neutron Time of Flight System	48

APPENDIX 1 PROTON RECOIL SPECTRUM DUE TO N-P SCATTERING  
IN ANTHRACENE

APPENDIX 2 NEUTRON DETECTION EFFICIENCY OF A BIASED  
SCINTILLATION COUNTER

APPENDIX 3 DATA SMOOTHING BY FOURTH DIFFERENCES

APPENDIX 4 COMPUTER PROGRAMS DEVELOPED FOR THE  
PRESENT INVESTIGATIONS

ACKNOWLEDGEMENTS

LIST OF REFERENCES

CHAPTER 1

INTRODUCTION

## 1.1 ALPHA DECAY STUDIES

Particle decays of excited states of the carbon-12 nucleus have been studied by several experimenters in recent years. Many different approaches have been adopted for populating levels in this nucleus. For a comprehensive survey the reader is referred to the compilation of energy level data by Ajzenberg-Selove and Lauritsen (1968).

Of particular relevance to the present study are experiments utilising inelastic neutron scattering on the nucleus of interest to provide the primary excitation. Nuclear emulsion studies have provided useful information on the channels used by the decays of the first few states of carbon-12 (see Perkin 1951, Frye et al. 1955). Here neutron bombardment of a film of carbon-rich emulsion was used to raise carbon-12 nuclei to excited levels. Charged particle decays produced visible tracks whose range and direction permitted identification of the nuclear reactions involved. Cases where the final state included three independent alpha-particles were identified by the formation of three-pronged "stars".

Inelastic scattering of neutrons from graphite has been used in conjunction with a neutron time of flight spectrometer by Barjon et al. (1962) and others, to yield differential inelastic scattering cross sections corresponding to individual final states of the recoiling carbon nuclei. Barjon's measurements were made for a primary neutron energy of 15 MeV; an interesting contribution was

found in the time of flight spectrum which could not be connected with levels in carbon, and it was attributed to the reaction,  $^{12}\text{C}(n,\alpha)^9\text{Be}^*(n)^8\text{Be}$ , with the intermediate beryllium-9 nucleus being excited to a level at 6.76 MeV.

A comprehensive study at 14.1 MeV has been undertaken by Joseph et al. (1967) using the same technique; these experimenters measured detailed angular distributions corresponding to the carbon-12 levels at 4.43, 7.65 and 9.63 MeV, correcting their results for multiple scattering in the sample by a Monte Carlo calculation, and using maximum likelihood theory to estimate errors due to background subtraction. Joseph et al. found a broad neutron group which could not be explained by inelastic scattering from carbon. It was suggested that this group was due to inelastic scattering from a broad level in carbon at about 10.1 MeV, in addition to various sequential decays of which the  $(n,\alpha)$  reaction was the first step.

The existence of this broad level was suggested by Peelle (1957) on the strength of results from inelastic scattering of 16.7 MeV protons from carbon. Confirmation of the excitation of carbon nuclei to this level has been sought by Grin et al. (1965, 1966, 1969) and others, but its existence still seems to be in doubt.

A new approach to particle decay studies of carbon-12 states has become possible with the advent of pulse shape discrimination systems used with organic scintillation detectors (Brooks 1956,

Owen 1958). Johnson (1963) has used a single scintillation counter to observe alpha-particles arising from the bombardment of a stilbene crystal by 14.85 MeV neutrons.

Grin et al. (1965) have devised an ingenious experiment, extending the neutron time-of-flight technique for use with organic scintillators equipped with pulse shape discrimination circuitry. The experiment involved neutron bombardment of a plastic scintillator, and permitted recording of differential cross sections for inelastic scattering from carbon nuclei for the first three excited states; some idea could also be gained of the branching ratios for electromagnetic and alpha-particle decays of these states by amplitude discrimination. Again, the neutron energy was 14.1 MeV.

The following section outlines the approach of Johnson (1963), as extended for the requirements of the present investigation, and briefly describes those properties of organic phosphors which make the approach possible.

## 1.2 ORGANIC PHOSPHORS

Phosphors find application in nuclear physics as charged particle detectors and, indirectly, as detectors of other ionising radiations which give rise to charged particles or ions, by reaction or scattering processes within the phosphor. In particular, in a study of properties of carbon nuclei, a carbon-rich phosphor may be used as both target and detector. Fast neutrons incident on a phosphor populate certain excited states of carbon nuclei

by inelastic scattering; a decay proceeding by charged particle emission produces a scintillation in the phosphor which may be amplified by means of a photomultiplier tube.

In the present study a crystal of anthracene is irradiated with monoenergetic fast neutrons; ideally it is desired to record only those scintillations induced by alpha-particles. There are two major sources of background signal to contend with. Many incident neutrons do not interact with carbon nuclei but are instead scattered by hydrogen nuclei within the phosphor, the recoil protons producing scintillations. Also there is a high flux of background gamma radiation produced by neutron interaction with surrounding materials, as well as by electromagnetic decays of carbon states within the phosphor itself, giving rise to electron-induced scintillations from gamma-ray-electron interactions.

It has been known for some time that in certain phosphors, a particle-induced scintillation is composed of an intense fast decaying and a slowly decaying component, so that the light output may be approximated by

$$L = A e^{-t/\tau_1} + B e^{-t/\tau_2}, \quad \tau_2 \gg \tau_1.$$

For the crystalline organic phosphor anthracene, Owen (see Birks, 1964) has determined  $\tau_1 \approx 33\text{ns}$  and  $\tau_2 \approx 370\text{ns}$ . The importance of this lies in the fact that the ratio B/A is dependent on the energy loss of the ionising particle per unit distance,  $dE/dx$ , which is in turn determined by the nature of the particle.

In anthracene  $B/A$  for a proton is given by Birks (1964) as approximately twice that for an electron. An alpha-particle induces an even higher slow to fast component ratio. This property has been taken advantage of by Brooks (1956, 1959), Owen (1958) and others, who have operated a photomultiplier tube with a low potential difference between the last dynode and the anode, so that an amplified pulse produces space-charge saturation in this region. In the Brooks circuit, a simple passive network mixes the outputs from the last dynode and the anode. The mixed output has greater amplitude if induced by a recoiling proton than if arising from an electron recoil, for equivalent integrated light outputs of the phosphor. Full details of the theory of operation of this circuit have been described by Brooks (1959).

This pulse shape discrimination (PSD) technique has been successfully applied to liquid scintillators too. For the liquid scintillator NE213 Henchoz (1965) has found  $\tau_1 \cong 2.5\text{ns}$  and  $\tau_2 \cong 150\text{ns}$ , with  $B/A \cong 0.7\%$  for electrons and  $\cong 1.4\%$  for protons.

There is an additional scintillation property, however, applicable only to crystalline phosphors; it is an asymmetry effect associated with the crystal axes. For anthracene, stilbene and certain other crystals, Tsukada and Kikuchi (1962) and others have shown that the height and shape of a scintillation pulse each depend on the direction of the particle trajectory with respect to the crystal axes. In the present study an anthracene crystal was placed at the optimum orientation (found by experiment) with

respect to a fast neutron beam for pulse shape discrimination outputs deriving from proton- and alpha- induced crystal excitations to be as markedly separated as possible.

The Brooks and Owen PSD circuits have been criticised on two accounts. Batchelor et al. (1960) have pointed out that the circuits producing the pulse shape discrimination signal do not remove the possible overshoot produced when the pulse returns to zero voltage. Detection of overshoots is obviated in the present investigation by the use of a gating circuit, described in Chapter 2.

de Vries and Udo (1961) have found it necessary to introduce an interference network into the Owen circuit to reject pile-up effects encountered in experiments involving high count rate. As the highest detected scintillation rate was 9.4 kHz in the present study, no such modification was required.

The initial energy of an electron in anthracene is proportional to the response (luminous intensity) produced in this phosphor. This holds true over an appreciable energy range. For heavier charged particles and ions, however, the relationship is not a linear one. Brooks (see Birks, 1964) has obtained these relationships for electrons, protons and alpha particles (see figure 1.2). Gooding and Pugh (1960) have suggested a design for a linear scintillation counter; no attempt has been made to construct such a counter for this work, and the curves of figure 1.2 have been used for quantitative calibrations.

Carbon nuclei were bombarded with neutrons of energy as great as 22 MeV in this investigation; figure 1.1 shows those excited

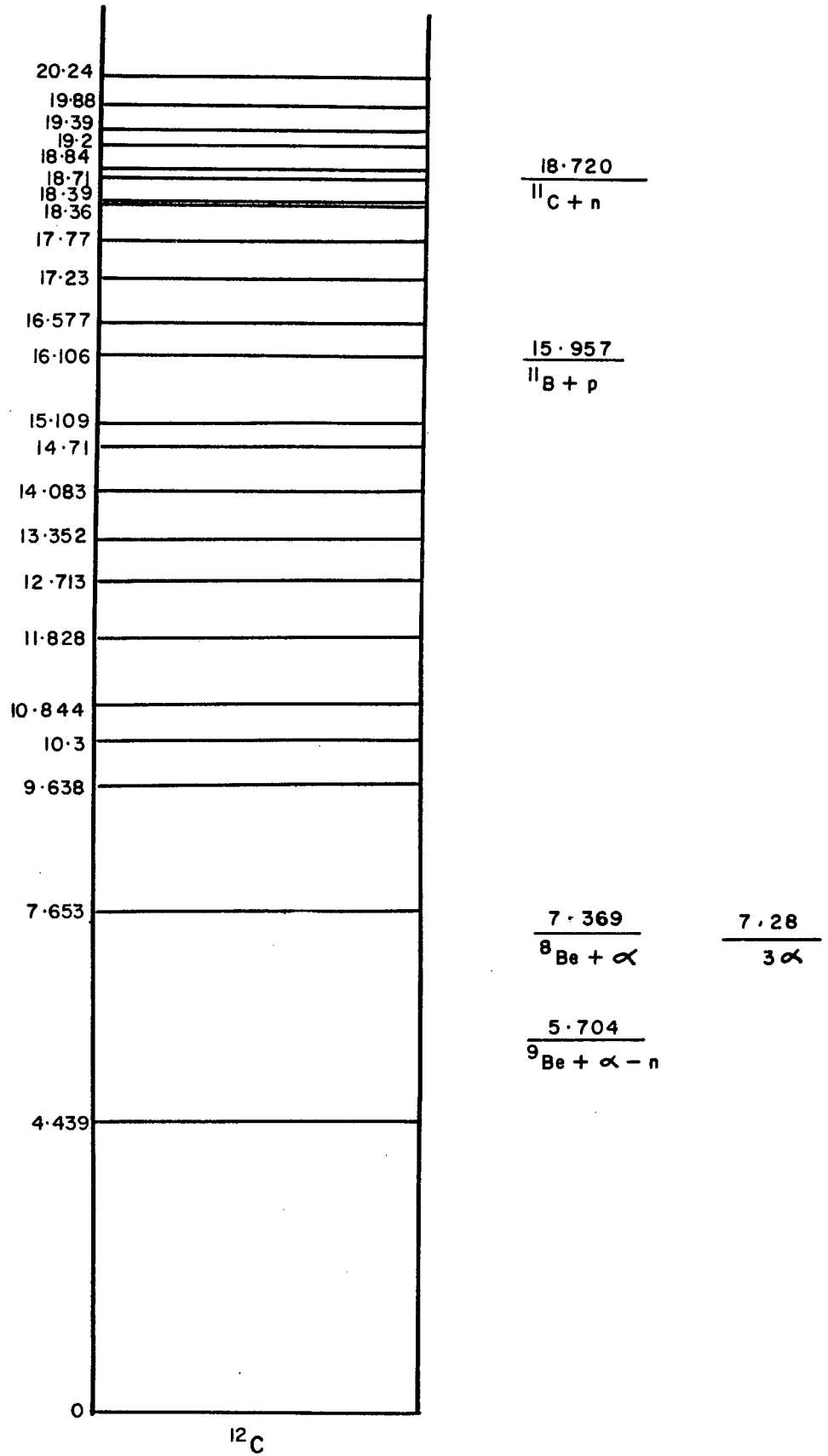


Fig.1.1 Energy levels of Carbon-12, in MeV.

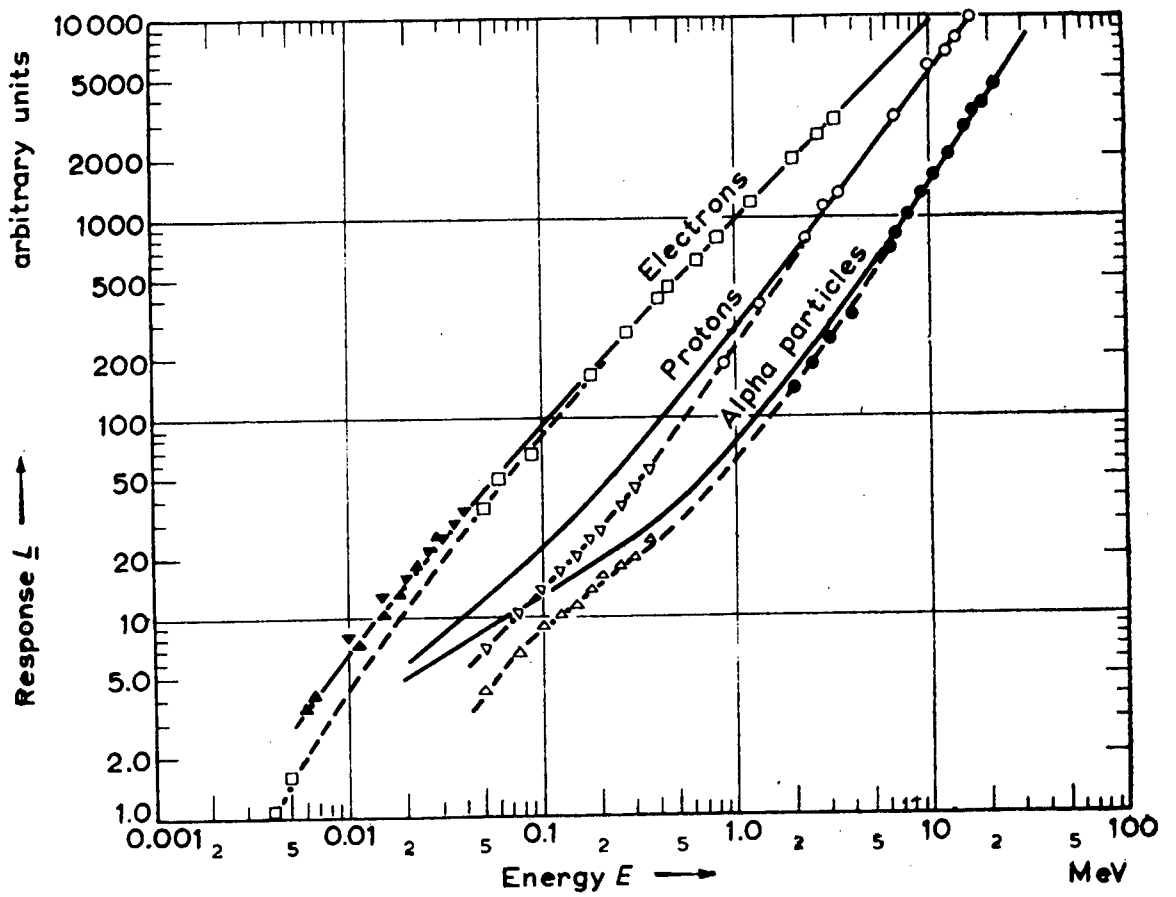


Fig. 1.2 Response of an anthracene crystal to electrons, protons and alpha-particles.

----- external radiation, measured.

————— internally generated radiation, calculated.

states which can be populated by neutrons of this energy (taken from Ajzenberg-Selove and Lauritsen, 1968).

While pulse shape discrimination may be applied to separate alpha-particle events from proton and electron recoil contributions, there will inevitably be a contaminating background of alpha-particles from the reaction  $^{12}\text{C}(n,\alpha)^9\text{Be}(\text{g.s.})$ , a process which sheds no light on decay of carbon-12 states.

### 1.3 OBJECTS OF THE PRESENT INVESTIGATION

The primary aims of the present study are to use the compact target-cum-detector approach to investigate the disintegration by alpha-particle emission of carbon-12 nuclei; also to study reactions arising from fast neutron bombardment of the organic hydrocarbon phosphor, anthracene. Furthermore it is intended to seek indications of the existence of an energy level at 10.1 MeV in carbon-12, searched for by Johnson (1963), Joseph (1967), Grin (1969) and others, and peculiar in that the width of this level appears to be some 2 MeV. It is also proposed to extend the effective resolution of proton and alpha-particle events in a scintillation counter down to low energies. Finally, in order to make comparisons with the results of previous authors, branching ratios for decay channels of individual states are to be sought; this requires additional information specifying the rates of population of states under fast neutron bombardment, for which purpose differential inelastic scattering cross-sections are to be measured using a neutron time of flight technique.

CHAPTER 2

EXPERIMENTAL ARRANGEMENTS

Detailed descriptions of apparatus used in this project are given in this Chapter for the three major experiments performed. Much preliminary laboratory work was carried out to study the variables in the Brooks PSD circuit and auxiliary electronic equipment, using radioactive sources, so that optimum discrimination conditions could be achieved; also timing resolution of a few nanoseconds was sought after.

Altogether four detectors were used. It was judged that the best pulse shape discrimination conditions for the present application were obtained with an EMI 6097 photomultiplier tube, and this was used where no timing measurements were necessary. A timing resolution of 3ns was obtained for two RCA 6810A tubes (used with a Chronetics model 151 "leading-edge" discriminator), one equipped with a PSD circuit and the other arranged to give a fast anode output and a linear 5th dynode output only. The fourth detector used in this project was built by Jones (1967) and used a Philips 58 AVP tube to provide 3ns timing resolution and  $\gamma$ -n discrimination.

## 2.1 NEUTRON PRODUCTION

In each of the experimental arrangements described below, a source of monoenergetic fast neutrons was obtained from the reaction  ${}^3\text{H}(d,n){}^4\text{He}$ , with the primary deuterons generated as a collimated monoenergetic beam by the S.U.N.I. 5.5 MV Van der Graaff.

accelerator. A tritium gas cell was used as the target for preliminary work; a tritium target consisting of one Curie of tritium adsorbed onto titanium on a gold backing was subsequently obtained and used for all measurements. This target was subjected to up to 3 microamperes of deuteron current at 5 MeV, and it was accordingly necessary to apply a cooling system to it.

A liquid nitrogen cold trap was coupled to the target by means of a stainless steel cold finger. A thermocouple was situated inside the cold trap at about half the maximum liquid nitrogen level. Whenever its temperature rose appreciably above the boiling point of nitrogen, this thermocouple activated a pumping mechanism which automatically refilled the trap. A chart recorder situated in the control area was used to monitor the temperature of this thermocouple. As a safety precaution, a second thermocouple was situated somewhat lower in the trap, and used to trigger an alarm bell.

The "solid" target had two distinct advantages over the gas cell, in addition to greater ease of handling. Firstly, the location of the neutron source was now accurately defined. Secondly, no window was required to separate the tritium from the high vacuum of the accelerator tube, and consequently a major source of deuteron energy reduction and spread was removed.

## 2.2 NEUTRON DETECTION

A crystal of anthracene was optically coupled to the face of a type EMI 6097 photomultiplier tube by a film of liquid paraffin. The size of this crystal was approximately  $1.5 \text{ cm}^3$ . The crystal surfaces not in contact with the photomultiplier were covered with a thin, reflecting aluminium foil to improve the efficiency of light-collection. This assembly was enclosed in an aluminium housing, held by a support which, while keeping the crystal and the axis of the photomultiplier tube in the horizontal plane defined by the target, permitted rotations of the detector about the vertical axis through the crystal (angle  $\theta$ ) and about the axis of the photomultiplier tube (angle  $\phi$ ). Provision was also made for rotation of the whole combination about a vertical axis through the tritium target (angle  $\gamma$ ). These angles are represented in figure 2.1. The incident neutron energy could thus be selected by appropriate choice of angle  $\gamma$ , while any axis through the crystal could be aligned with the incident neutron direction by variation of angles  $\theta$  and  $\phi$ . A boron "long-counter" which viewed the neutron source directly was set up as a neutron monitor. This counter comprised a  $\text{BF}_3$  detector surrounded by a cylindrical paraffin wax modulator.

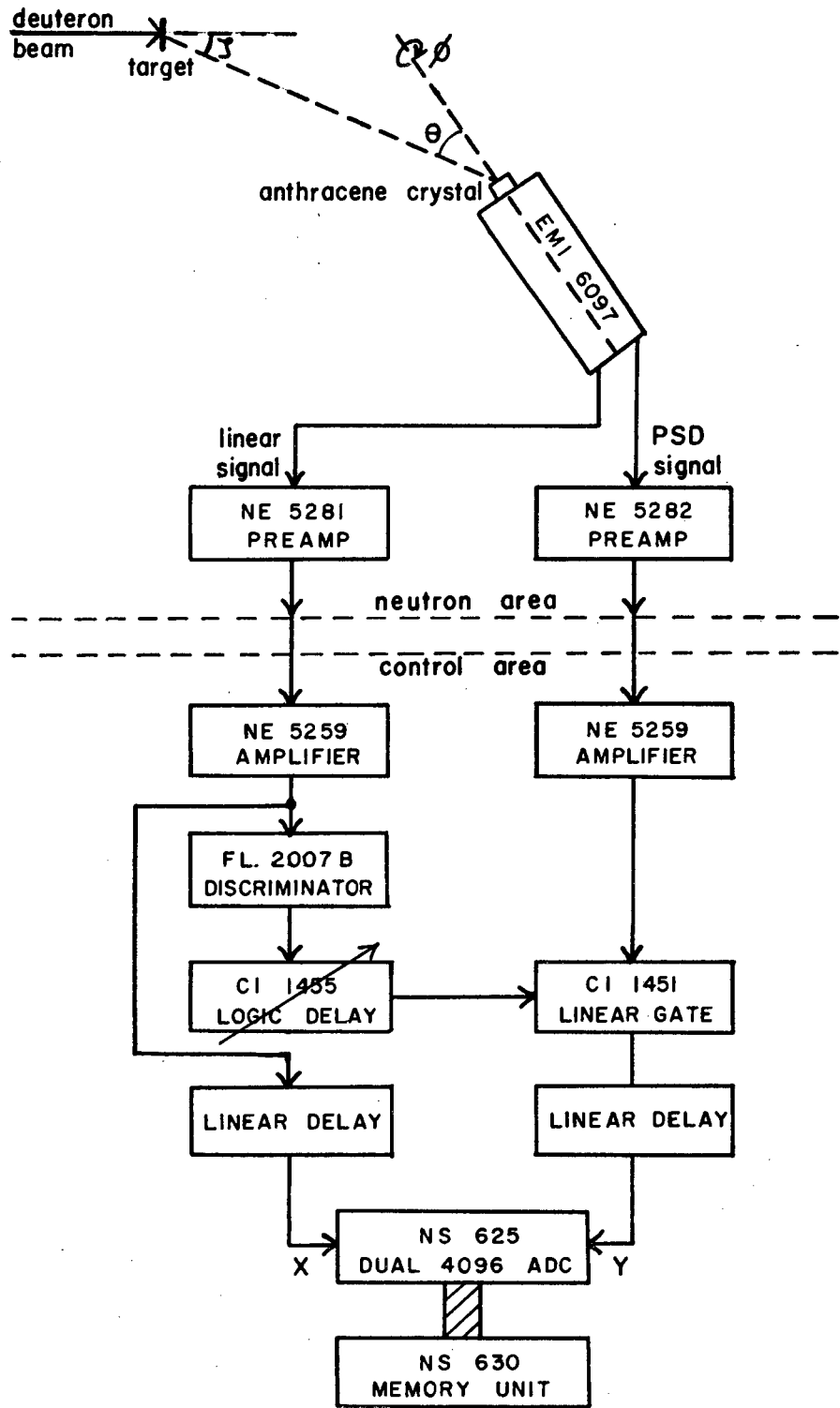


Fig. 2.1 Block diagram illustrating angles  $\gamma$ ,  $\theta$  and  $\phi$ , and showing circuit used to select time of observation of PSD signal for two-parameter analysis.

### 2.3 PULSE SHAPE DISCRIMINATION SYSTEM

A pulse shape discrimination circuit of the type developed by Brooks (1956, 1959) and described previously in Section 1.2 was mounted within the aluminium detector housing (see figure 2.2).

The output pulses from the PSD amplifier for excitations due to electrons, protons and alpha particles differ in amplitude most significantly at some time  $t$  after the initial rise, and a circuit was used to interrogate the PSD output at this time (see figure 2.1), producing a "gated PSD" signal. The width of the gating signal was limited to reject overshoots (Batchelor et al., 1960).

In the present study the PSD technique of particle identification has found two distinct applications. The simpler of these is the now conventional application, where an amplitude discriminator is applied to the gated PSD signal and adjusted to produce a logic pulse if a neutron was detected, but no pulse in the case of an incident gamma ray. This application is used to convert a standard scintillation counter into a neutron counter, and is employed in the present investigation as the detector in a neutron time of flight spectrometer (see Section 2.5 and Chapter 4).

Pulse shape discrimination has been used here in another way. The technique has been applied not only to eliminate gamma ray background, but to distinguish alpha-particle events from proton events.

The amplitude of the PSD signal from the photomultiplier tube is dependent both on pulse height and pulse shape, and therefore

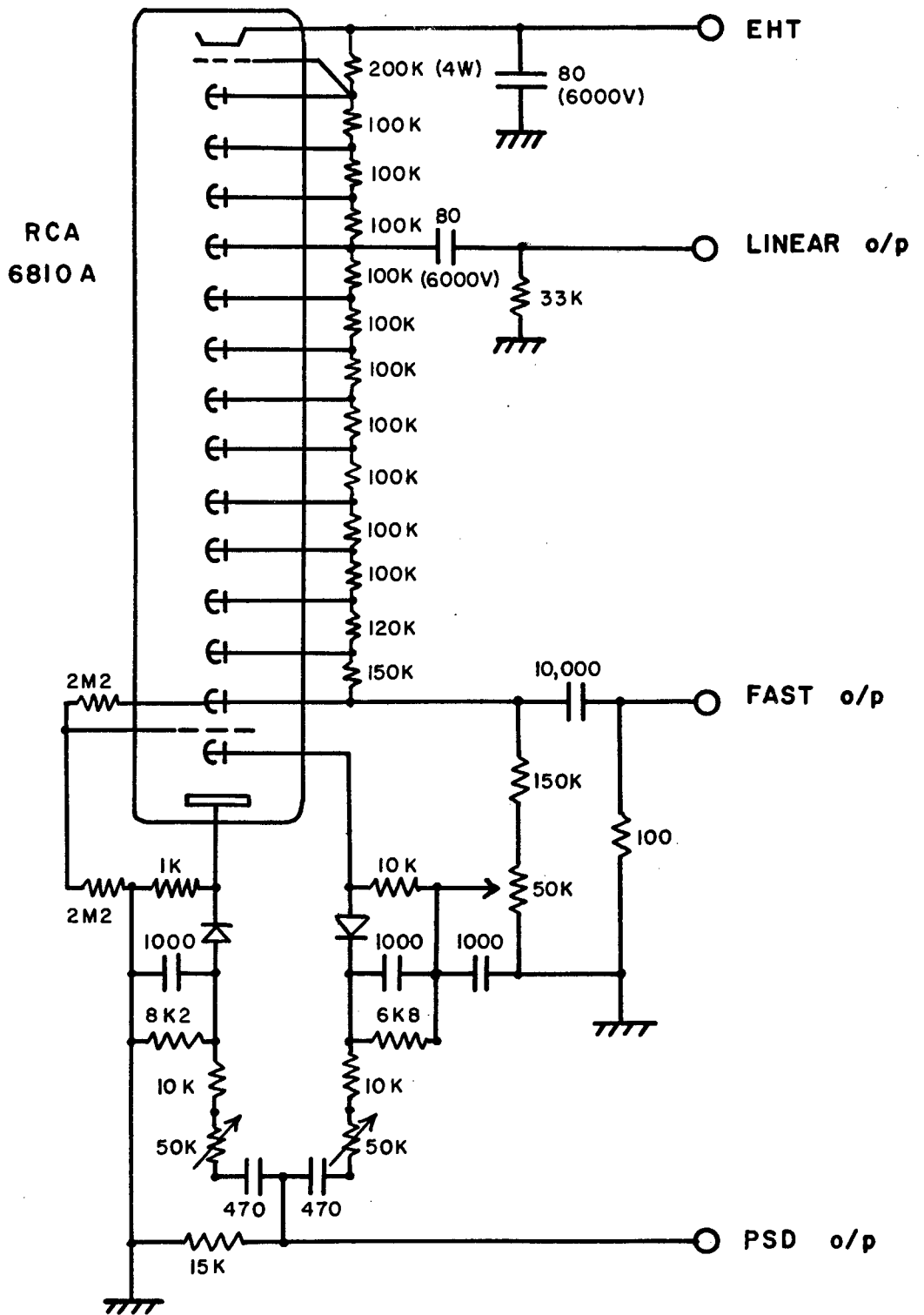


Fig.2.2

Pulse shape discrimination system used with an RCA 6810 A photomultiplier tube, the whole being enclosed in the detector housing.

on the energy as well as the nature of the exciting particle. We may therefore construct a plot of the amplitude of the gated PSD signal as a function of energy, and we will obtain one curve for each type of particle considered. In the present experiment this process has been carried out electronically, in that the linear and gated PSD signals were aligned in time using suitable microsecond delays, and fed to the X and Y inputs of a Northern Scientific dual parameter analog to digital converter. (See figure 2.1). The two parameter digital information generated by each scintillation was recorded in a Northern Scientific 12-bit Memory Unit, the 4096-channel capacity of this unit being utilized as 128 channels for linear pulse amplitude and 32 channels for the gated PSD pulse amplitude. The contents of the Memory Unit could be displayed by a built-in oscilloscope in contour and isometric formats. Figure 2.2A has been traced from a photograph of the contour mode oscilloscope readout, and demonstrates the dependence of the gated PSD signal on both energy and particle type. The data were also punched onto computer cards (automatically in some cases) and printed out by a parallel printer.

It was necessary to calibrate the energy axis of the 32 x 128 element matrix of channels, and this was performed as follows. The detector was exposed to gamma-rays from a cobalt-60 source in place of fast neutrons, and a 2-parameter spectrum was recorded with the PSD amplifier gain boosted so that the electron-recoil locus occupied a prominent region of the matrix. The gain of

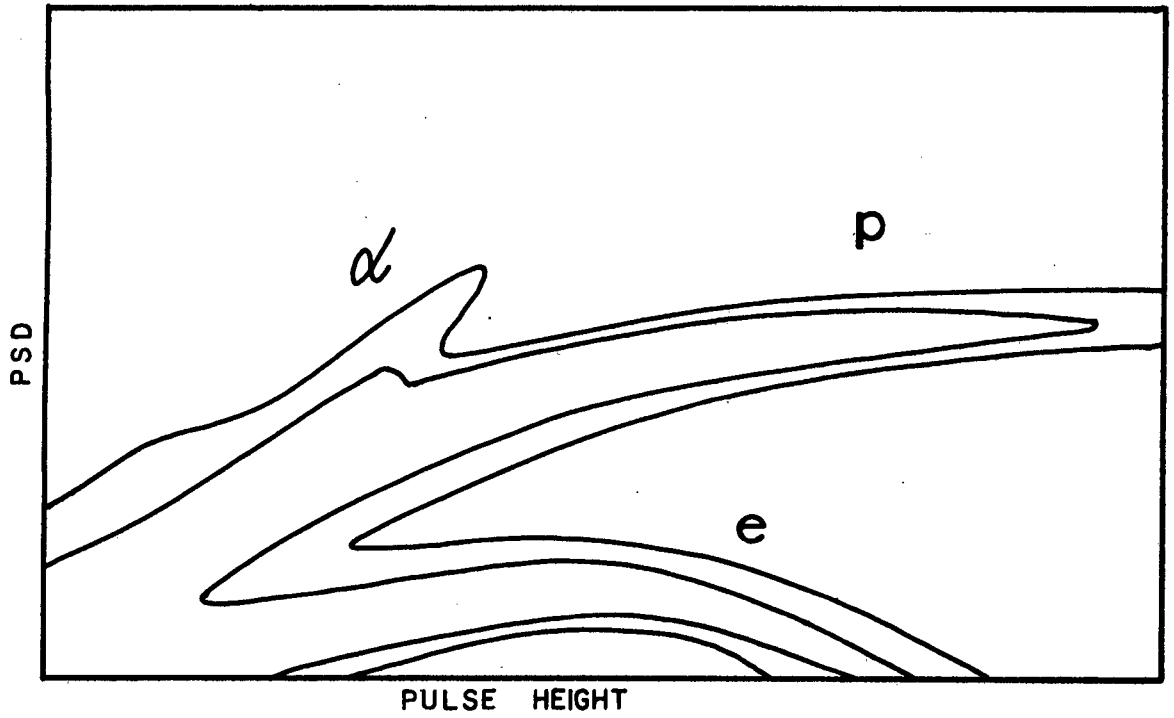


Fig. 2.2A Pulse shape discrimination amplitude as a function of energy and particle type.

the linear signal amplifier was then increased by a factor of 2.5 and a second spectrum superimposed on the first. (See figure 2.3.)

Integrations over PSD channel number in the region of the first locus yielded the characteristic electron energy spectrum induced by Co-60 gamma rays; the Compton "edge" of the highest energy is attributed to 1.33 MeV gamma rays, and the position of this edge in the spectrum therefore corresponds to an electron energy equal to the maximum recoil energy attained by a Compton recoil electron from a 1.33 MeV gamma ray, viz. 1.12 MeV. A comparison of the two loci indicates the channel of zero pulse height. The axis is now calibrated in terms of electron energy, and hence also proton and alpha energies, from the response curves for anthracene (see figure 1.2).

Experiments carried out with this system, together with the results obtained, are described in Chapter 3.

#### 2.4 COINCIDENCE SYSTEM

While the alpha and proton loci obtained with the dual parameter system described above are well resolved at high energy, the separation becomes less clearly defined in the region of low light output (discussed in Chapter 3). In view of this an extension of the above experimental system was devised in order to suppress the recording of n-p scattering data.

The EMI 6097 tube was replaced by an RCA 6810 A tube. The wiring of these two tubes was essentially the same. A second

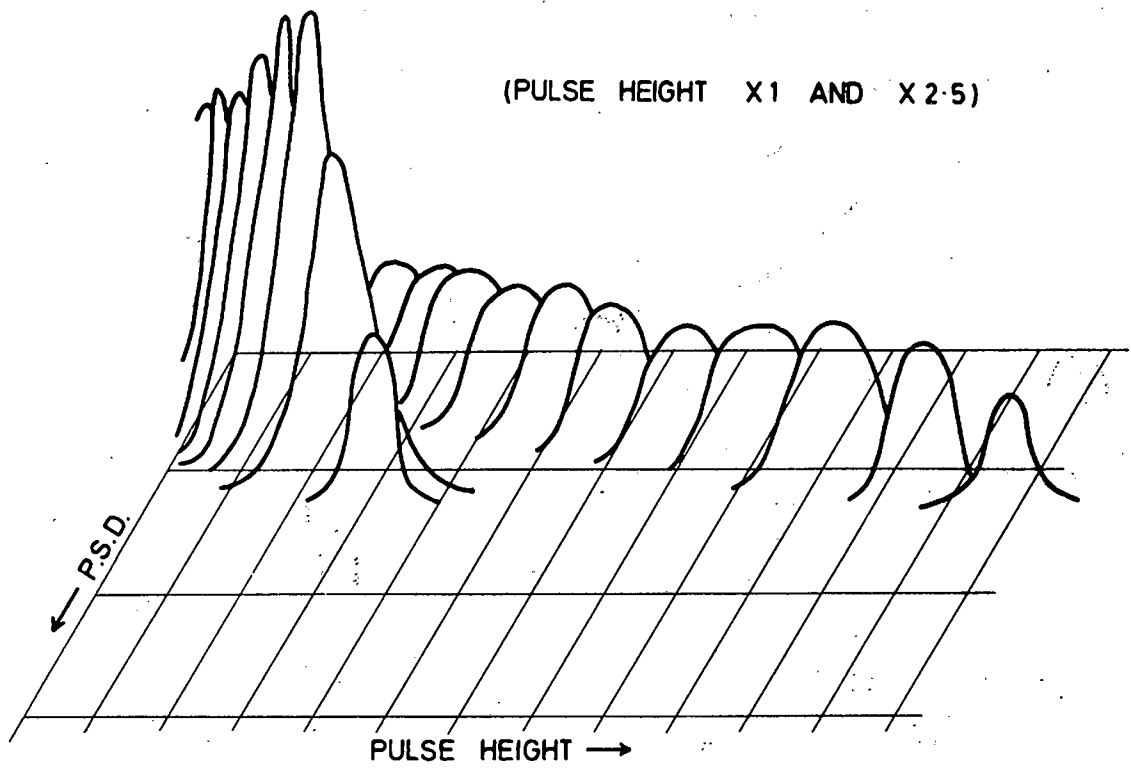


Fig. 2.3 Loci due to Co-60 gamma rays.

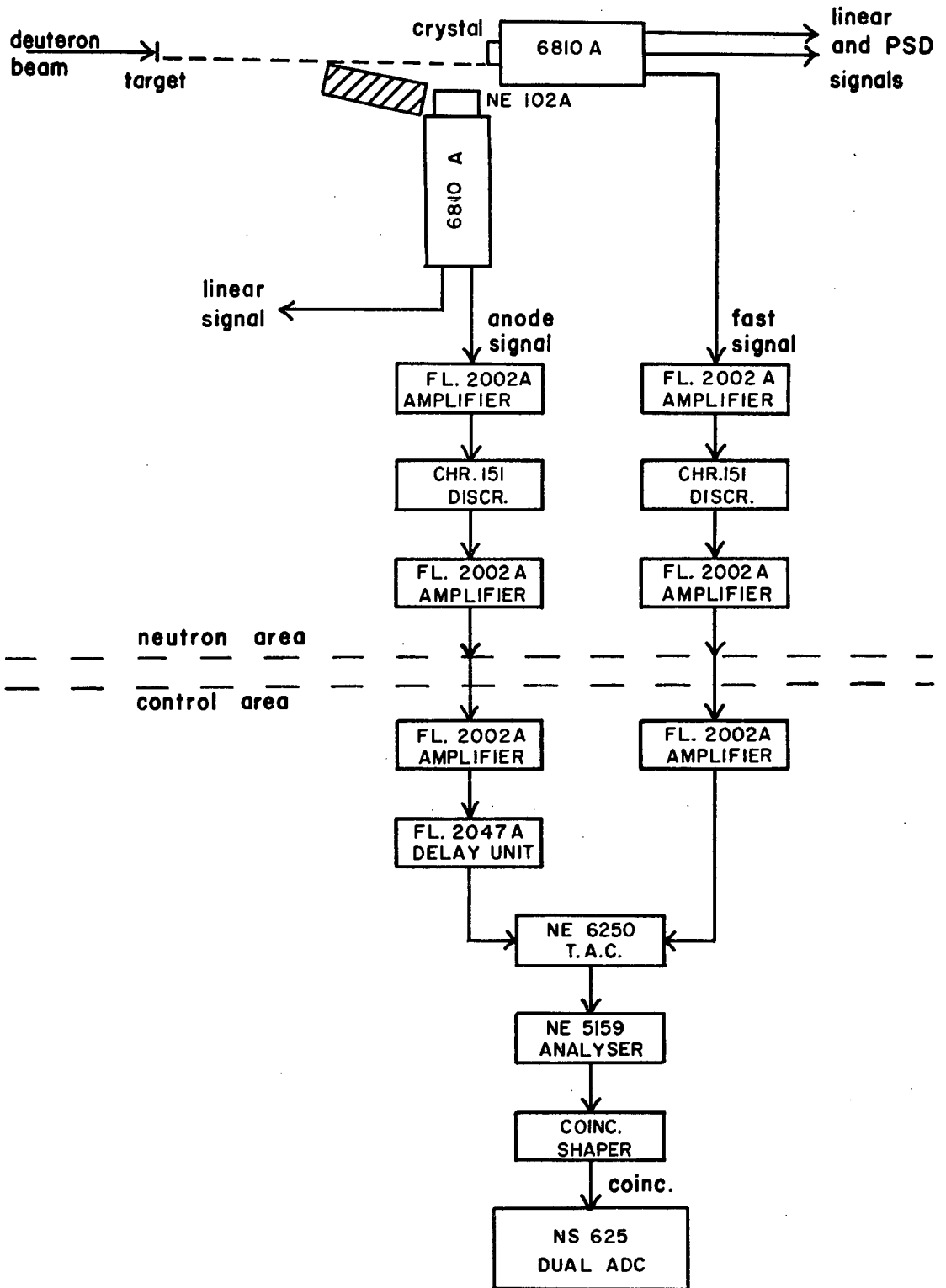


Fig. 2.4 Fast coincidence arrangement for suppression of n-p scattering data acquisition. The circuit forms an auxiliary to that of figure 2.1.

The circuit of figure 2.4 was used to generate a fast coincidence signal when a scintillation occurred in the plastic phosphor within approximately 10 ns of a scintillation in the anthracene crystal. Linear and gated PSD pulse heights originating in the crystal were recorded only for those events in coincidence with this signal.

Data were accumulated in sets each consisting of three experimental runs. The first run used the coincidence arrangement described above to record true coincidences. For the second run the 10 ns acceptable range of neutron flight times was deliberately misaligned to record random coincidences. In the third run the coincidence requirement was removed (switch-selectable on the dual ADC) and two parameter information was stored whether a coincidence signal was present or not.

As before, a boron long-counter was used to monitor the neutron flux leaving the target.

#### 2.4A CALIBRATION PROCEDURE

Calibration of the scattered-neutron coincidence system was carried out in four stages as follows.

##### i) Fast bias setting - crystal

A Caesium-137 source was placed close to the anthracene crystal, and the amplified linear signal was fed through a delay unit to the Northern Scientific analyser, operated in single parameter mode, to produce a pulse-height spectrum. A coincidence signal was derived from the leading-edge fast discriminator output

and fed to the analyser, and coincidence spectra were recorded for various settings of the fast discriminator bias control until an appropriate threshold was reached (see figure 2.5). A comparison of the positions of the low energy cut-off and the high energy falling edge of the Cs-137 electron-recoil spectrum (478 keV electron energy) determine the bias level as 160 keV electron energy, the channel of zero pulse height having been determined as -4 (due to digital gates in the dual ADC). The proton detection threshold is therefore (figure 1.2) 640 keV, which is equivalent to an effective alpha-particle energy of 1.9 MeV.

ii) Fast bias setting - plastic

A linear pulse was derived from the 5th dynode of the photomultiplier coupled to the plastic scintillator, and the procedure outlined in i) above was carried out for the second detector. The detection threshold obtained corresponded to 120 keV electron energy. The results of Batchelor et al. (1961) for scintillation response in NE 102 have been used to give the corresponding proton energy as 520 keV. The neutron detection threshold is therefore also 520 keV, considering n-p scattering as the sole detection mode (at keV neutron energies this is a good approximation).

iii) Selection of coincidence resolving time

The two detectors, with fast signal bias levels now fixed, were placed so that the phosphors were immediately on either side of a Sodium-22 source. Annihilation gamma rays produced in pairs

and leaving the source at  $180^\circ$  to one another gave rise to simultaneous scintillations in the phosphors. The output of the time-to-amplitude converter shown in figure 2.4 was fed to the multi-channel analyser to generate a time spectrum which was recorded in coincidence with a signal derived from the window discriminator driven by the time-to-amplitude converter output. The position and width of the window were adjusted so as to accept scattered neutron flight times up to approximately 10ns. The window width as narrow as possible was chosen to enhance the signal to background ratio of recorded spectra.

Peak A of figure 2.6 was accumulated with the "stop" signal delay as used for recording true coincidence data, while peak B was recorded with the calibrated "stop" signal delay decreased by 9.85ns. It is seen from the rapid fall to zero on the left of peak B that this peak is close to the lower level of the window.

As the flight path was  $3.7 \pm 2.7$  cm, all scattered neutron velocities above about 0.64 cm/ns would be accepted by the window. This velocity corresponds to an energy of 220 keV, which is well below threshold.

The timing resolution of the two biased detectors is seen from peaks A and B to be 5ns for 0.51 MeV gamma rays. This high value is attributed to the fact that a considerable fraction of recoil electrons have energies close to the respective detection thresholds, a situation which, with leading-edge discrimination, inevitably leads to significant jitter. It is expected that the

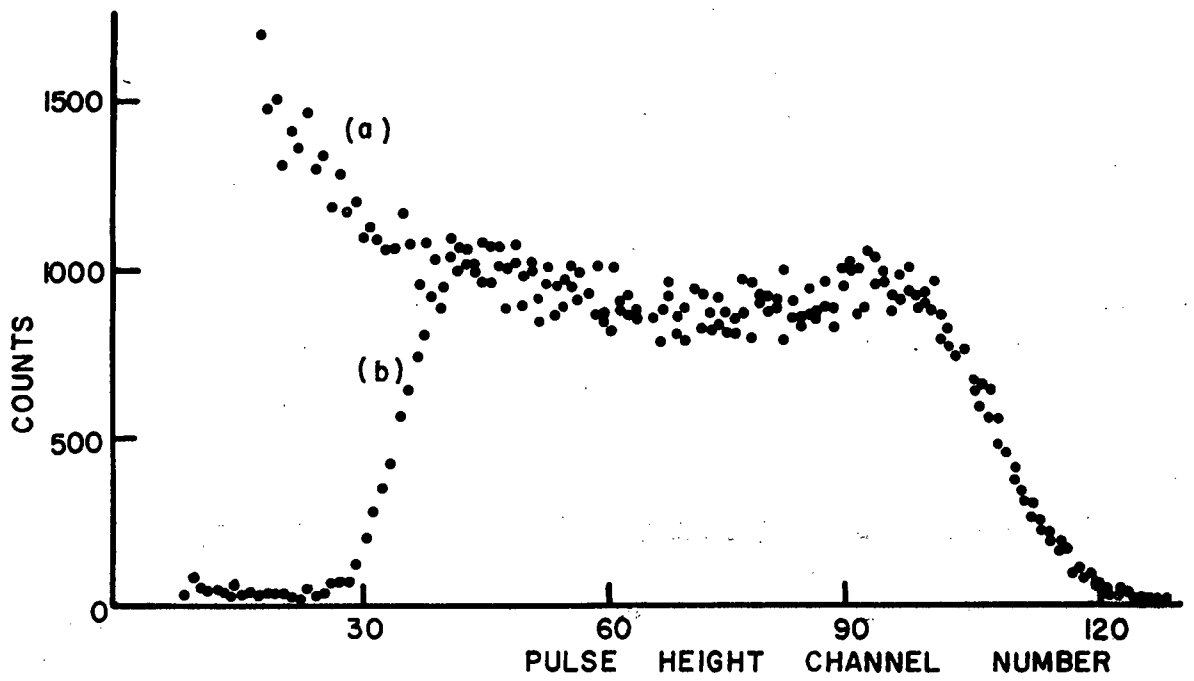


Fig. 2.5 Overlap of Cs-137 pulse height spectra in anthracene  
 (a) not gated, (b) gated by fast discriminator output.

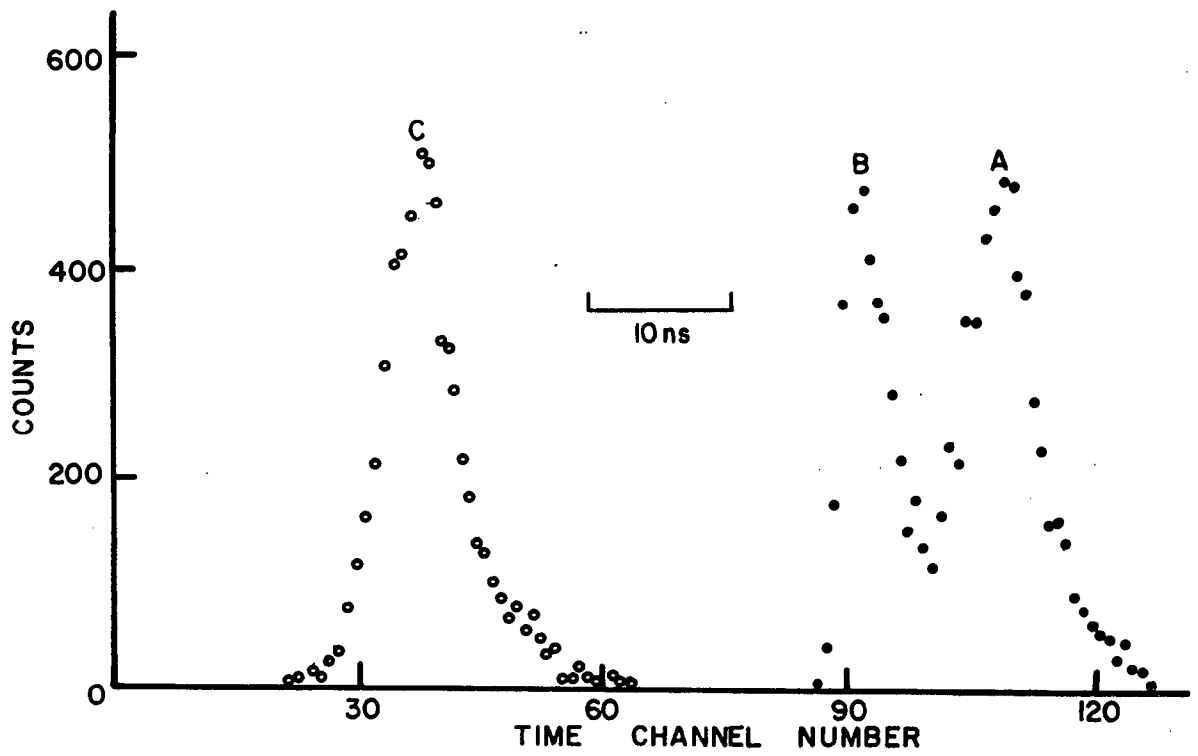


Fig. 2.6 Time spectra for annihilation gamma rays from a Na-22 source

resolution for neutrons of energy greater than about 2 MeV will be better than this; nevertheless some loss of counts must occur due to the choice of window width in relation to the resolution.

The "stop" signal delay was decreased by 39.18ns and a gated spectrum recorded. No counts were accumulated. The coincidence requirement was then removed to record peak C of figure 2.6. This value of the "stop" signal delay was adopted as appropriate for measurement of spectra due to random coincidences.

By selecting a different delay rather than adjusting the position of the window, possible alteration of the window width was avoided.

#### iv) Determination of energy scale

The dual parameter system of Section 2.3 was assembled and an energy scale calibration carried out using a Co-60 source in the same manner as described in Section 2.3.

With the Na-22 source back in position between the phosphors, the linear and gated PSD pulses deriving from the anthracene crystal, and the coincidence signal, were aligned in time using a twin-beam oscilloscope, and the three pulses were fed to the appropriate inputs of the Northern Scientific dual parameter analyser. The detectors were then arranged as in figure 2.4.

The experiments performed with this system and the results obtained are described in Section 3.3.

## 2.5 NEUTRON TIME OF FLIGHT SYSTEM

In an attempt to determine the cross sections for population of individual excited states of the carbon-12 nucleus in the above experiments, a neutron time of flight spectrometer was assembled. A pulsed deuteron beam was allowed to impinge on the tritium target described above, giving rise to short bursts of neutrons at 500ns intervals. The pulsing was accomplished within the high voltage terminal of the Van der Graaff accelerator. The low energy (few hundred eV) deuteron beam leaving the ion source passed between deflection plates connected to a 2 MHz oscillator, arranged so that the beam was swept in an ellipse. In executing one full ellipse, the beam passed over a small aperture in a plate which otherwise blocked the beam. The transmitted deuteron pulse entered a bunching unit designed by Ortec, whose operation was synchronised to the arrival of the pulse. The bunched pulse was then accelerated by the high voltage of the machine.

A graphite cylinder of length 5.83cm and diameter 2.20cm was suspended by long, fine steel wires from a rigid aluminium support, so that it was a few cm in front of the tritium target, in the horizontal plane defined by the target. A pivot assembly bolted to the floor was positioned directly beneath the graphite cylinder, and a heavily shielded neutron detector was mounted on a trolley and constrained by a rigid steel arm to move in a circle with the pivot as centre. The neutron detector was adjusted to be in the horizontal plane of the target. The detector consisted of a 9.3cm diameter NE213 liquid scintillator mounted on a Philips 58AVP

photomultiplier tube. (See figure 2.7.) A PSD circuit identical to that shown in figure 2.2 was mounted within the aluminium housing enclosing the detector.

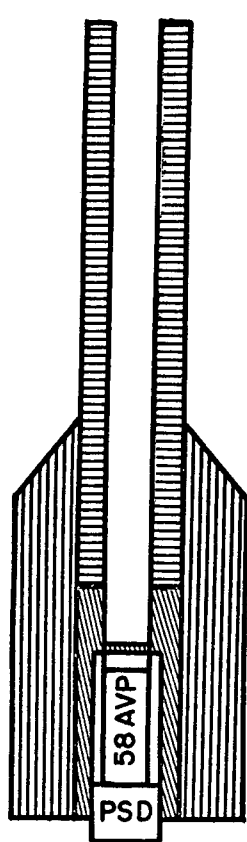
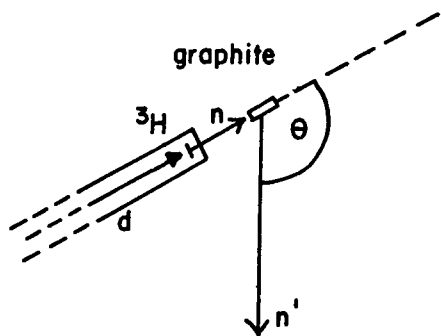
A circuit was assembled to measure the energies of neutrons scattered elastically and inelastically by the graphite, by recording their time of flight from the tritium target via the graphite to the liquid scintillator. This time interval was determined by the instant of arrival of a deuteron pulse at the target (detected by an inductive pick-off situated a few cm before the target), and the instant at which a scintillation occurred. A time-to-amplitude converter was used in order that a time of flight spectrum could be recorded in a conventional multichannel analyser. Flight times were recorded only in coincidence with a neutron identification output generated by the pulse shape discrimination circuitry employed. (See figure 2.8.)

#### 2.5A CALIBRATION PROCEDURE

The preliminary steps carried out to convert this system into a quantitative measuring device were as follows.

##### i) Fast bias setting

The beam pulse repetition time was only 500ns. It was therefore essential to set the overall neutron detection threshold high enough to reject pulses arising from neutrons whose total time of flight was in excess of about 400ns. Consideration of the flight paths from the target to the graphite cylinder, and from there to the detector suggested a desirable threshold of 300 keV.



50cm.



Paraffin Wax



Par. Wax +  $\text{Li}_2\text{CO}_3$  (1:1)



Lead

NE 213  
(4cm x 9cm diam.)

Fig. 2.7 Detector shield construction. Adapted from Jones (1967).

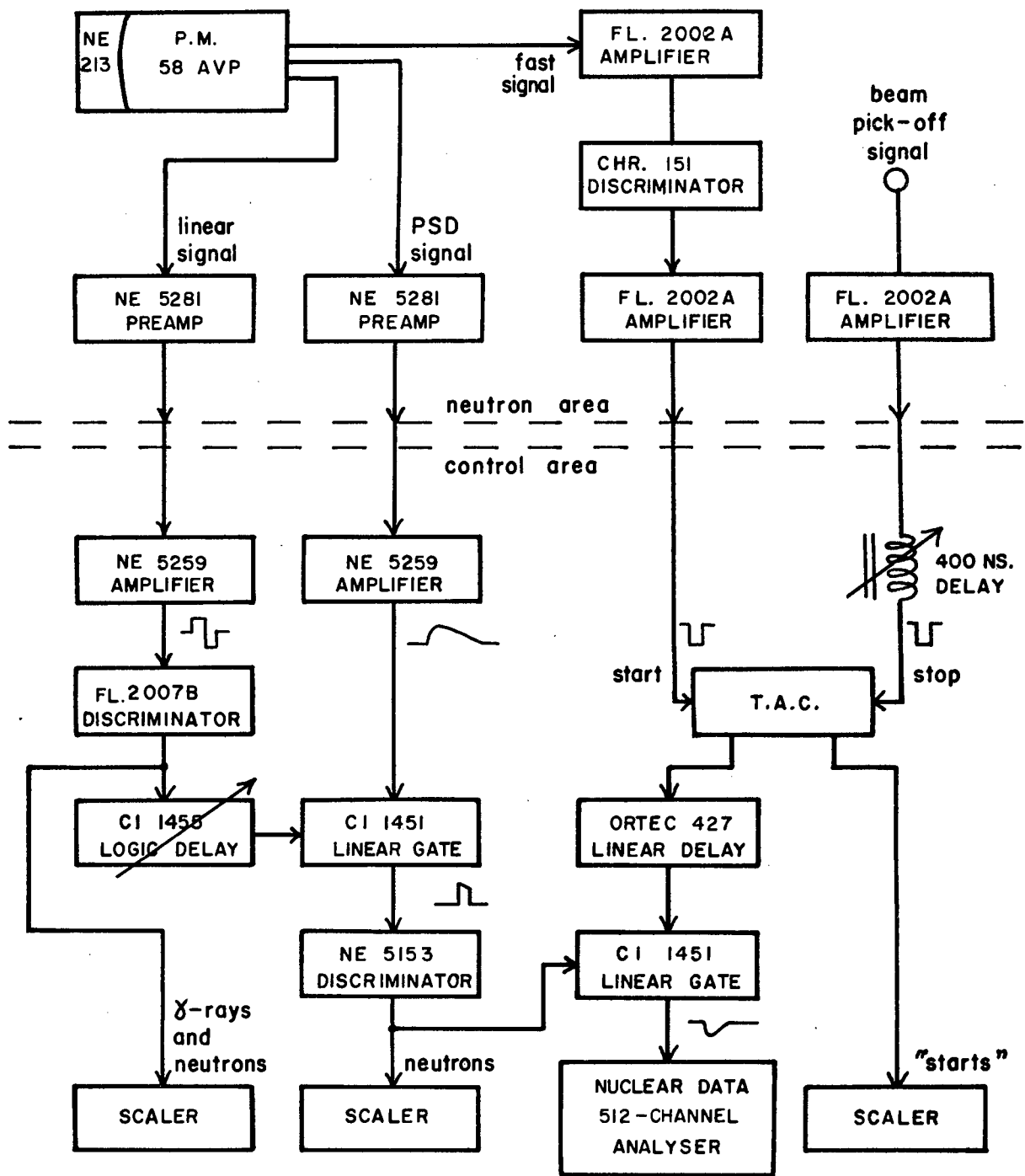


Fig. 2.8 Block diagram of electronic circuit used for  $^{12}\text{C}(n,n')^{12}\text{C}$  neutron time of flight measurements.

The bias level of the fast discriminator was set and recorded using a Cs-137 source. Gated and ungated pulse-height spectra were recorded by a procedure equivalent to that described in Section 2.4A (i). The result is presented in figure 2.9. As the neutron detection efficiency is a sensitive function of the detection threshold in the vicinity of threshold, a correction must be made in the low energy region. This matter is treated in Appendix 2.

ii) Gamma ray discrimination

The pulse shape discrimination part of the system was set up to show electron and proton loci due to an Am-Be neutron source, using a dual pulse-stretcher and X-Y oscilloscope (the Northern Scientific analyser described previously being unavailable). The output of the gated PSD lower-level discriminator (figure 2.8) was used to provide an external "bright-up" signal, and the discrimination level was adjusted so that the entire electron locus was not brightened. After this visual setting-up, pulse-height spectra of a Co-60 gamma ray source were accumulated in a Nuclear Data multichannel analyser, ungated and also gated by the output of the PSD discriminator, and final adjustments were made to the discrimination level so that a gated Co-60 spectrum was just indistinguishable from (gated) background.

The effective neutron detection threshold introduced was calculated from gated and ungated Am-Be pulse height spectra, by comparison with the Am-Be energy spectrum measured by Notarrigo

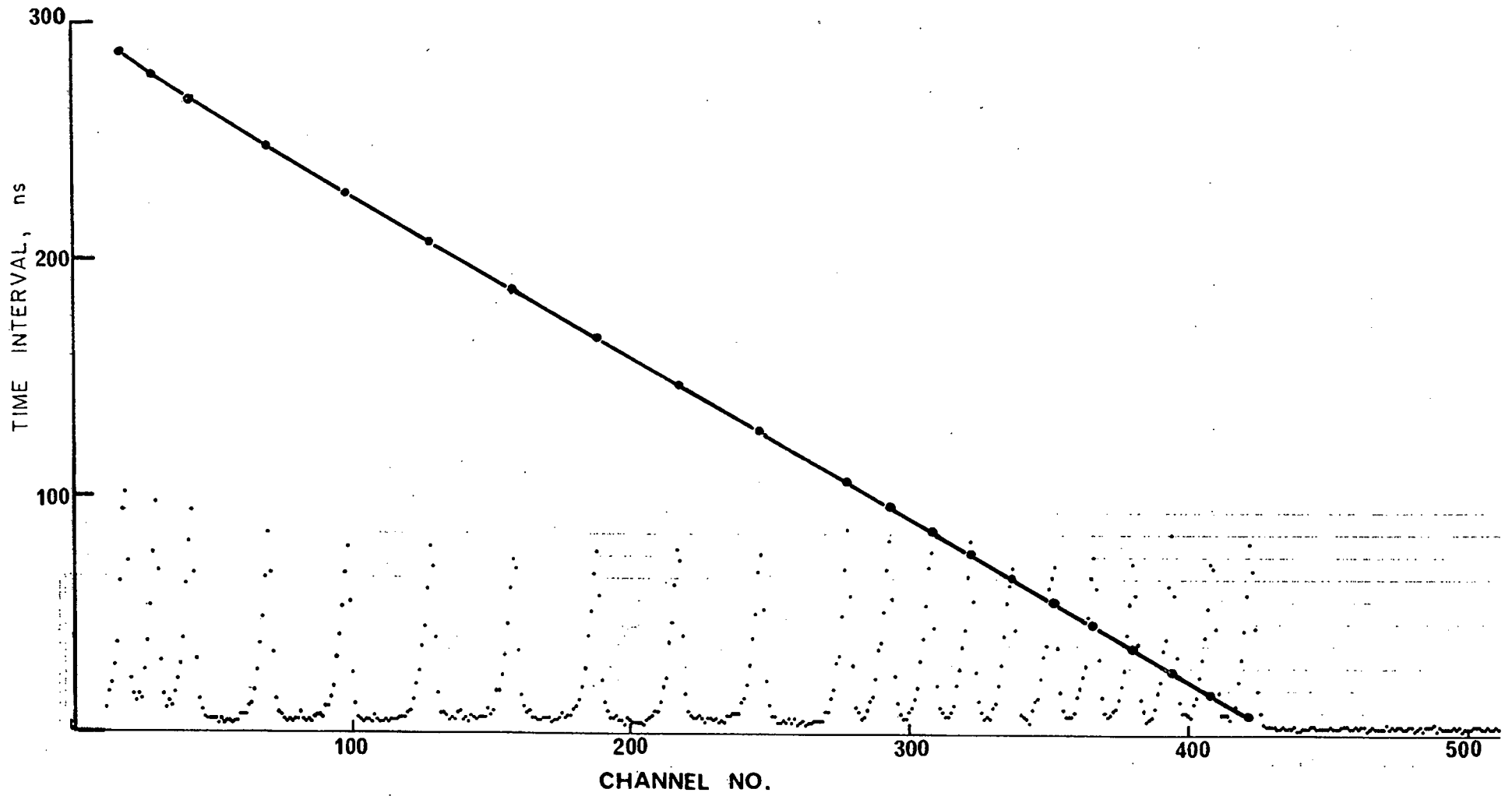


Fig. 2.10 Time scale calibration

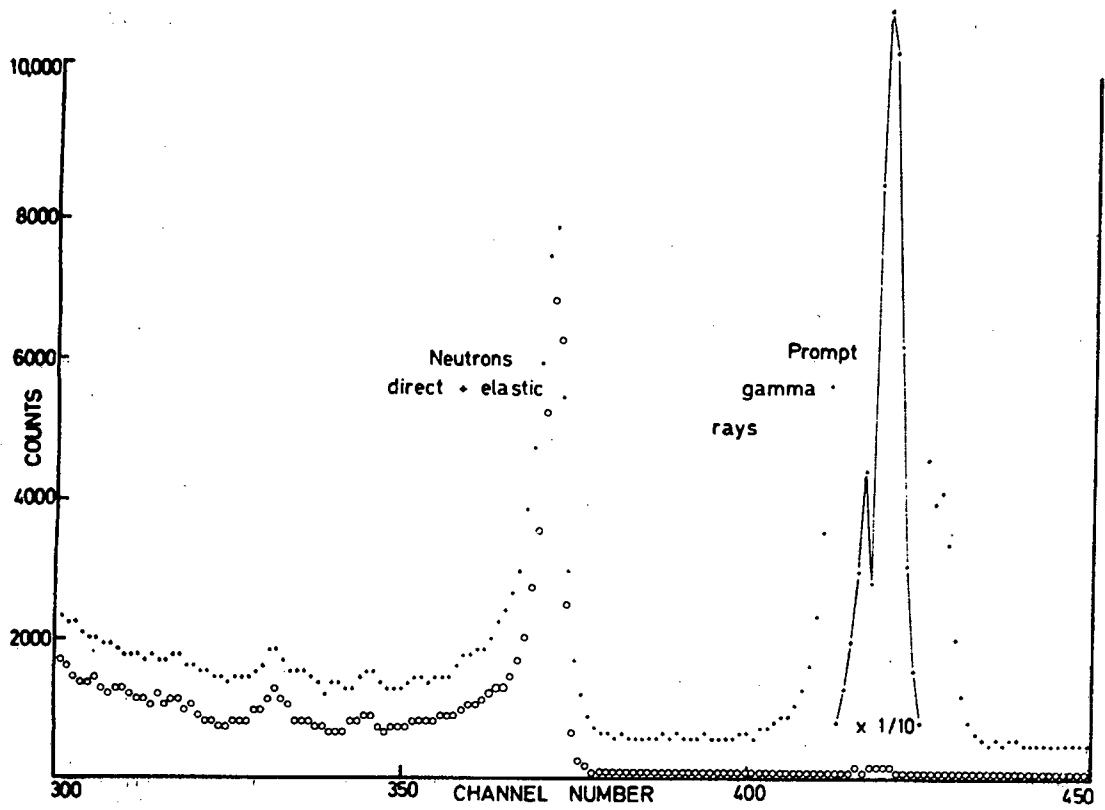


Fig. 2.11 Time of flight spectra. Upper curve, PSD off. Lower curve, PSD on.

the case of 14 MeV neutrons incident on a graphite block of volume more than four times that of the present scatterer. This group found that multiple scattering accounted for less than 20% of the total counts. Comparing the scatterer sizes, it is reasonable to assume that the contribution from multiple scattering in the present case was well below 5% of the number of singly scattered neutrons.

Details of the experiments carried out with this neutron time of flight system are presented in Chapter 4.

CHAPTER 3

MEASUREMENT OF ALPHA SPECTRA

Descriptions are given in this Chapter of the various techniques that were used to obtain pulse height spectra for alpha-particles alone, from data accumulated in fast neutron bombardments of a  $1.5\text{cm}^3$  anthracene crystal. Within the limits imposed by the particle resolution that was obtained using pulse shape discrimination, an analysis of the results is carried out.

### 3.1 THE DUAL PARAMETER EXPERIMENT

The experimental system described above in Section 2.3 was assembled to record two parameter spectra arising from fast neutron bombardment of anthracene. The photomultiplier tube was of the type EMI 6097.

Extensive preliminary experimentation was carried out at 22 MeV in order to examine the dependence of the linear and PSD pulse-heights on the crystal orientation with respect to the cone of neutrons incident on it. Two parameter spectra were displayed on the oscilloscope of the 128 x 32 channel analyser. The results agree qualitatively with the findings of Tsukada and Kikuchi (1962) for 3.7 MeV neutrons. The asymmetry properties of anthracene were found to affect the alpha-particle locus but little, while the proton recoil locus was seen to undergo significant shifts. The orientation  $\theta = 0$  was adopted as that giving the best separation of these two loci.

Energy resolution was limited by the few (128) channels

allocated to its analysis, while only 32 channels were available to accommodate the variation of PSD amplitude. Accordingly, amplifier gain settings and digital gates on the dual parameter multichannel analyser were adjusted to give great prominence to the alpha-particle locus. At this stage the energy scale calibration procedure described in Section 2.2 was carried out.

The energy of the deuteron beam was set at 5.000 MeV, corresponding to a nuclear magnetic resonance frequency of 29.162 MHz in the field monitor of the Van der Graaff's  $90^\circ$  analysing magnet, and was estimated to vary by less than 5 keV throughout the experiment. The beam current was maintained at approximately 1.5 microamperes.

Data were recorded for bombardments at angles  $\gamma = 0^\circ, 45^\circ$  and  $60^\circ$ . In each case the crystal was 18cm distant from the tritium target. The beam current was integrated electronically to yield the total charge reaching the target; the first of these runs was terminated when  $10^4$  microcoulombs had been counted (taking approximately two hours), while for the second and third runs twice this charge was recorded. The angles given correspond to neutron energies at the phosphor of 22.00, 20.05 and 18.78 MeV respectively, as calculated with the aid of the University's ICT 1301 computer, taking the Q-value for the  ${}^3\text{H}(d,n){}^4\text{He}$  reaction to be 17.58 MeV and using the non-relativistic formula. The values were checked against curves calculated by Goldberg (1963) and found to be in exact agreement.

The contour plot of figure 3.1, derived from the output of

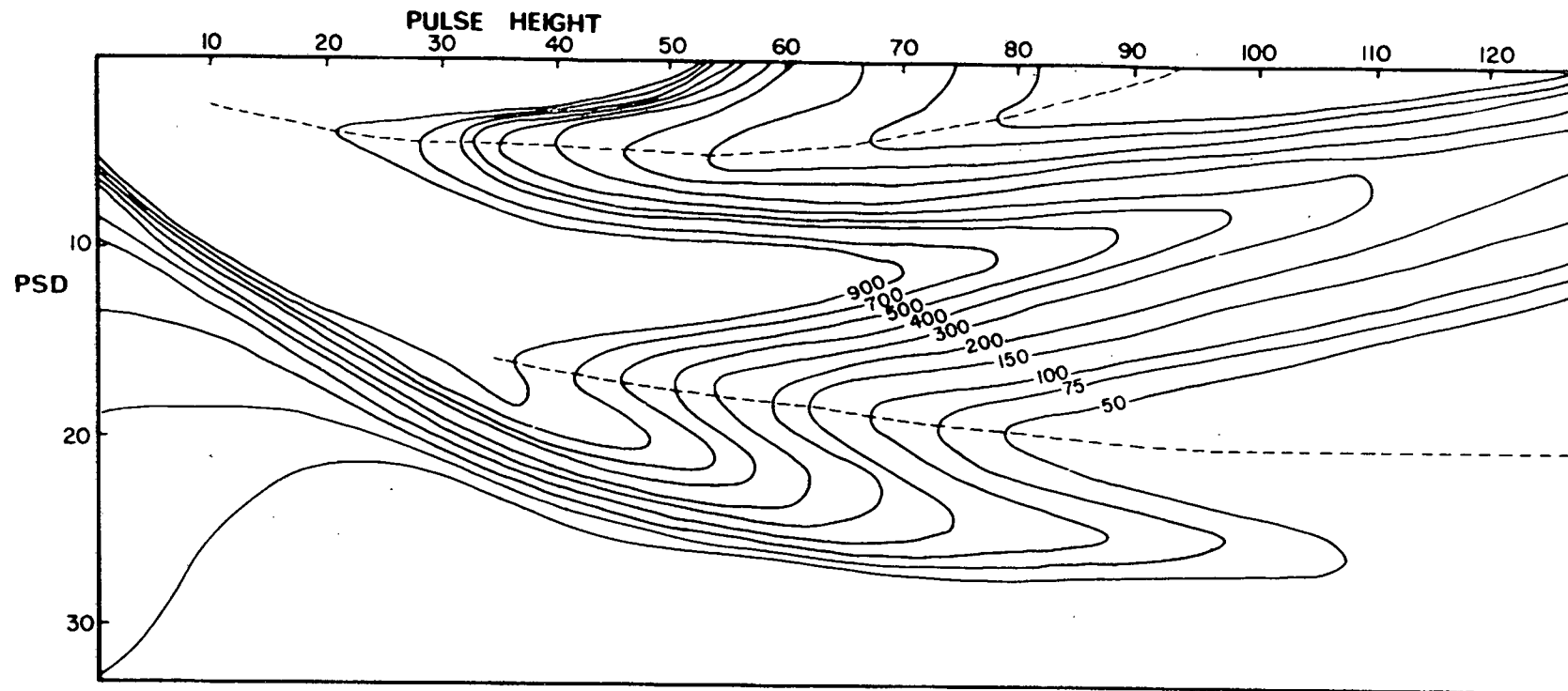


Fig. 3.1 22.0 MeV NEUTRONS ON ANTHRACENE  
CONTOUR PLOT — COUNTS vs. PULSE HEIGHT vs. PSD

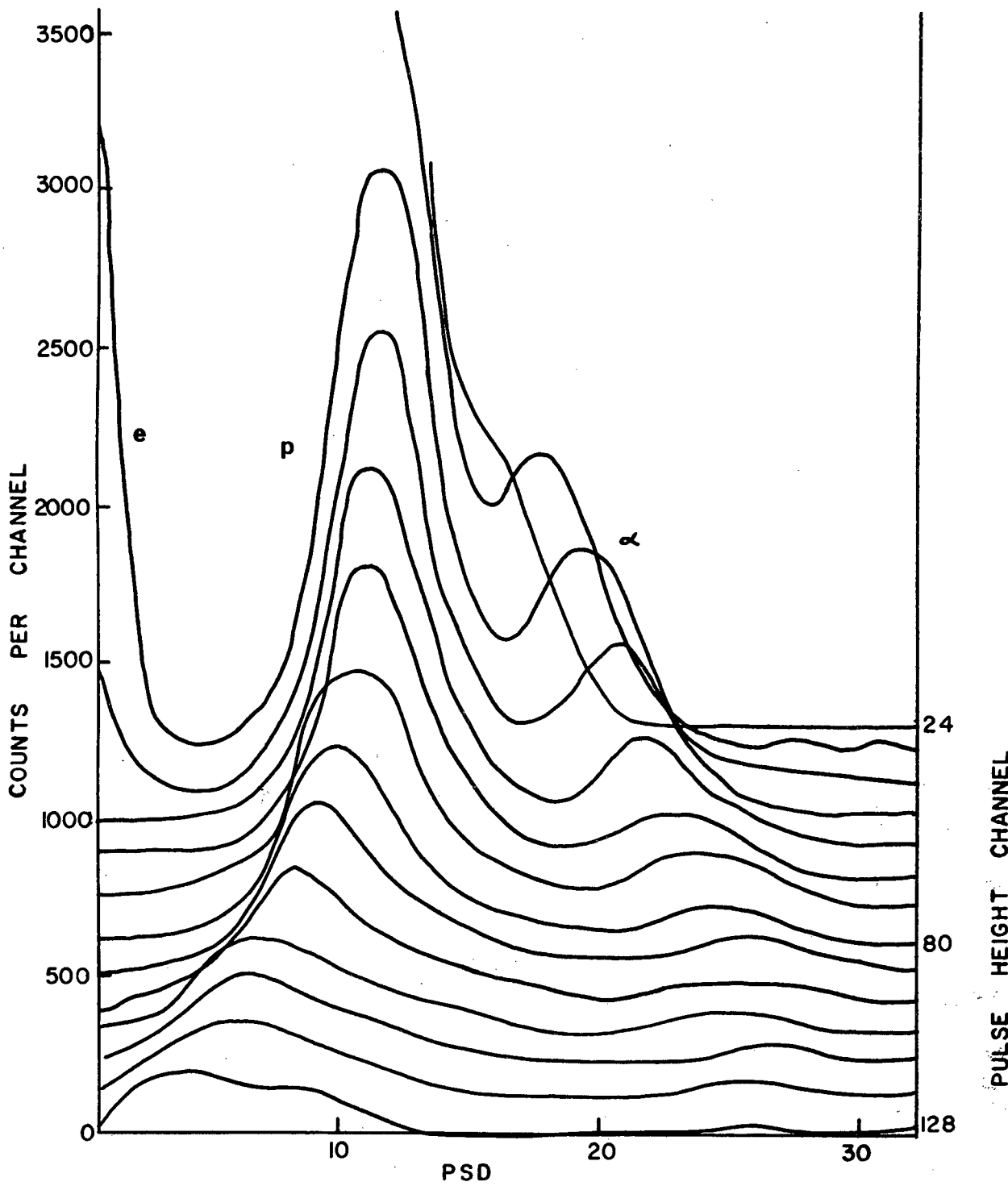


Fig. 3.2 PSD spectra at every eighth pulse height channel. Successive spectra are displaced by 100 counts per channel.

proton spectrum (calculated in Appendix 1) is monotonic. Structure obtained by Johnson was attributed to alpha decays of the C-12 states at 7.65 and 9.63 MeV, and to the  $^{12}\text{C}(n,\alpha)^9\text{Be}$  reaction.

In order to remove the proton contribution from the combined proton plus alpha-particle spectrum, two independent approaches were adopted. One was purely numerical, while the other involved subtraction of the calculated theoretical n-p scattering contribution. The numerical approach will be considered here, and the results of both methods discussed in Section 3.3.

It appears (figure 3.2) that towards the low energy region, the proton contribution at each energy channel takes the form of a PSD distribution that is a symmetrical peak. Then the proton contribution may be estimated by calculating the integral of proton counts from the electron-proton minimum up to the proton peak, and multiplying by a factor of 2. The validity of this technique rests heavily on the assumption that the proton contribution at each energy channel is large compared to the respective alpha-particle contribution, an assumption which is seen from figure 3.2 to be well-founded. In the presence of a large number of counts per channel in the region of a peak only a few channels wide, it was essential to determine accurately the position of the peak as a fractional channel value.

A computer program was developed for the University's IBM 1130 computer to implement this technique as follows (program ALPHA, listed in Appendix 4). The stages i) to v) were carried out for the PSD spectrum at each energy channel.

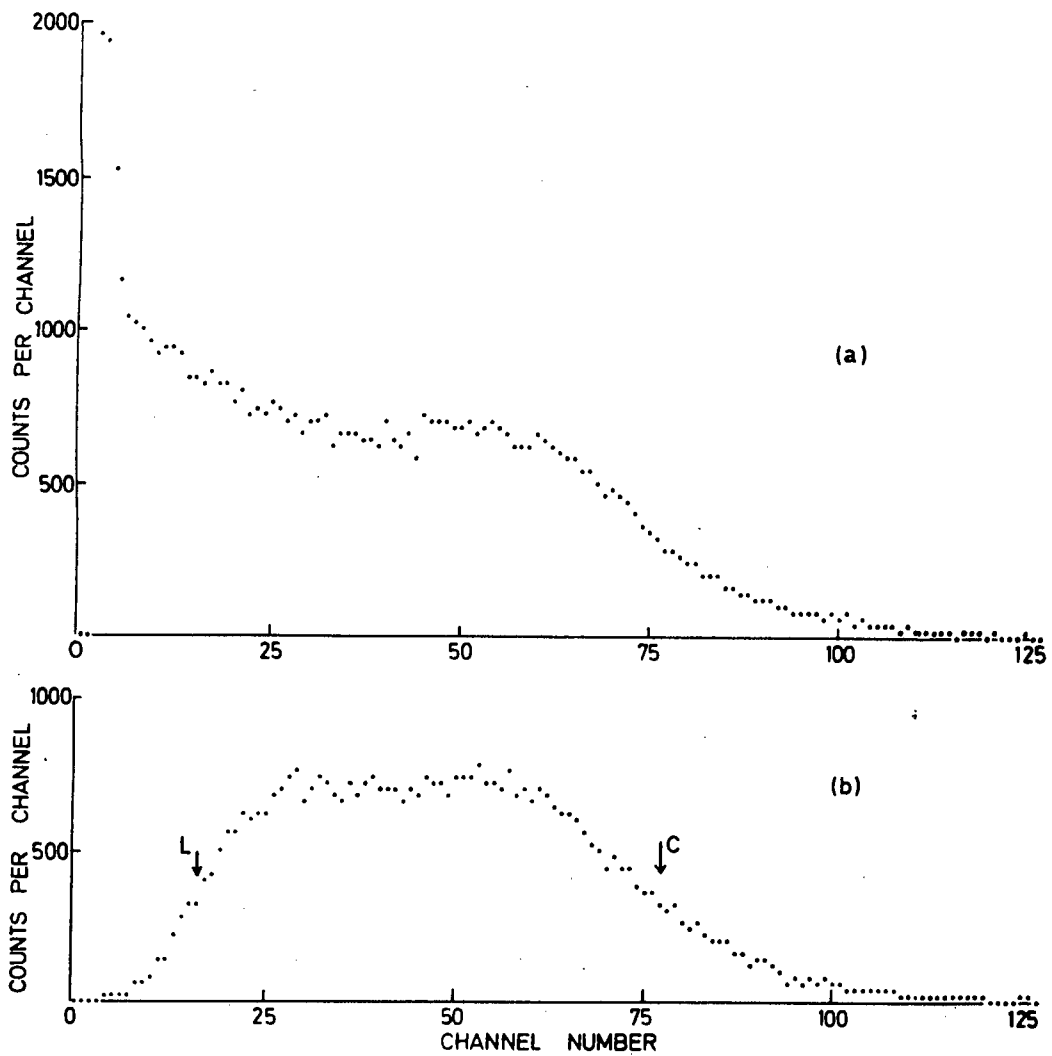


Fig 2.9 Caesium - 137 pulse height spectra (a) not gated, (b) gated by signal from fast discriminator, showing bias level L in relation to Compton edge C arising from Cs-137 gamma rays.

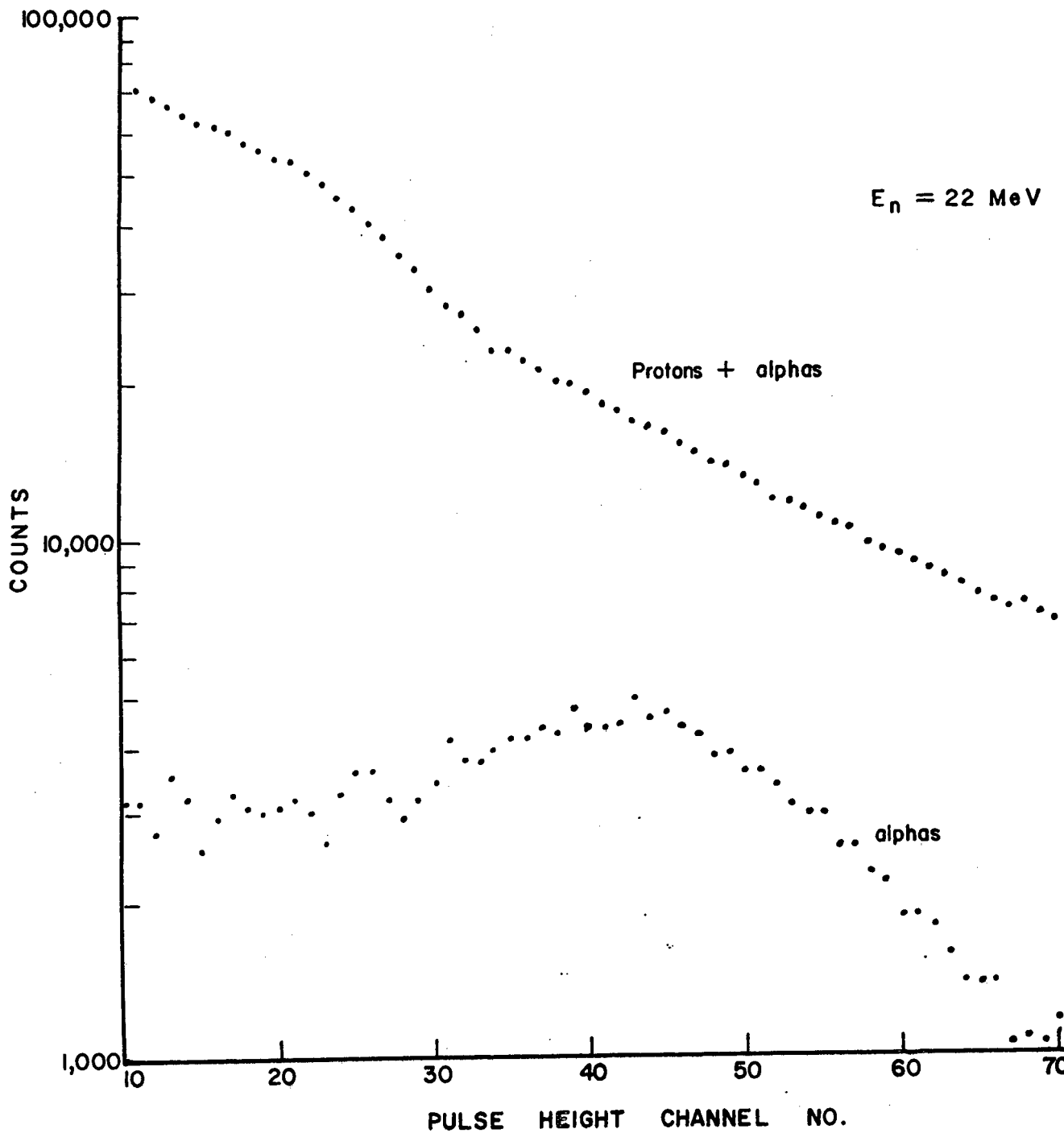


Fig. 3.3 Pulse height spectra generated by the program ALPHA.

Alpha spectra alone are shown in figure 3.4 for the three runs at 22.00, 20.05 and 18.78 MeV respectively. The derivation of the energy calibrations has been described in Section 2.2.

### 3.3 DISCUSSION OF RESULTS

Figure 3.5 shows a comparison between the proton spectrum as derived numerically from the data and that expected (see Appendix 1) from the n-p differential scattering cross section and the non-linear response of anthracene (figure 1.2), for the 22 MeV case. This discrepancy may be due in part to the reaction  $^{12}\text{C}(n,p)^{12}\text{B}$ . The transition to the ground state of boron-12 has a Q-value of -12.6 MeV and gives rise to protons of energy between 5.3 and 9.2 MeV in the laboratory frame, while excited final states correspond to protons of lower energy, in the region of the observed high count-rate; the cross section for this reaction, however, decreases from 29mb at 17.5 MeV to 13mb at 19.8 MeV (Jessen et al., 1966), and comparison of these values with the n-p scattering cross-section at 22 MeV suggests that there must be a further source of intermediate energy protons. The  $(n,p)$  reaction on carbon-12 ( $Q = -15.96$  MeV) may play a part, the highly energetic gamma rays required arising from electromagnetic C-12 de-excitations as well as from a general gamma ray background. It is likely that an important contribution stems from  $^{12}\text{C}(n,n')^{12}\text{C}^*(p)^{11}\text{B}$  reactions. Ten of the known carbon-12 states (Ajzenberg-Selove and Lauritsen, 1968) that can be populated by inelastic scattering at 22 MeV

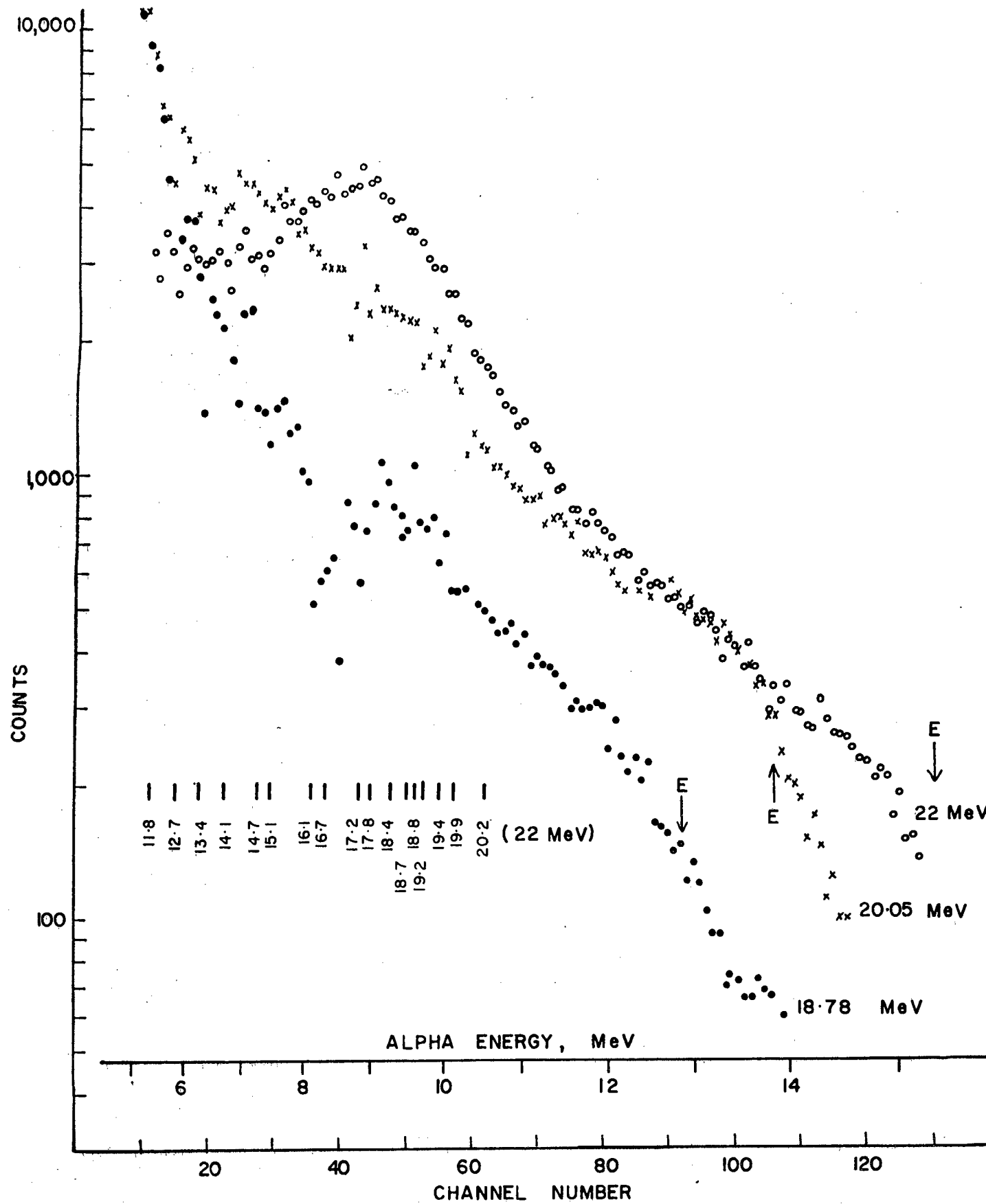


Fig. 3.4 Alpha spectra in anthracene.

have sufficient energy to decay by proton emission, and according to Segel et al. (1965) eight of these have an appreciable proton decay width compared to total width. This reaction mode may produce protons of energy up to 5.57 MeV. It is possible, too, that the beam of neutrons striking the phosphor is contaminated by a low-energy component for which the cross section for scattering from hydrogen nuclei is large (for the energy dependence of this scattering cross section see e.g. Enge, 1966). Such slower neutrons could be due to imperfect focussing of the primary deuteron beam, giving rise to neutron-producing interactions at the tantalum collimators or at one of the magnets on the deuteron line. Because of the uncertainty of the origin and significance of the high proton count-rate at low energy, it was decided to abandon the idea of using alpha-particle spectra obtained from subtraction of the theoretical uncontaminated n-p proton spectrum, and to consider only the numerically generated spectra.

The alpha-particle spectra of figure 3.4 each exhibit complex structure. In order to trace the origin of individual contributions, calculations have been performed for the  $3\alpha$ -disintegration process of each known state in carbon-12, to determine the positions on the spectra at which respective contributions might be expected. For inelastic neutron scattering leading to a particular state of the target nucleus, kinematical considerations indicate the minimum and maximum energies the scattered neutron may have, and hence the maximum and minimum recoil energies,  $T$ , of the target. In

the event of the nucleus disintegrating in flight, this recoil energy is added to the characteristic energy of the decay, the whole being distributed as kinetic energy among the three alpha-particles.

The form of the energy distribution may be estimated from the results of phase space arguments. Making the assumption that the disintegration  $^{12}\text{C}^* \rightarrow 3\alpha$  does not involve the intermediate nucleus  $^8\text{Be}$ , Frye et al. (1955) have obtained the energy distribution of any one alpha-particle in the carbon-12 centre of mass frame as

$$P(E_\alpha) dE_\alpha = \text{const} \cdot E_\alpha (E_{\text{max}} - (3/2)E_\alpha)^{1/2} dE_\alpha$$

where  $E_{\text{max}} = E_{\text{ex}}(^{12}\text{C}) - 7.28 \text{ Mev}$ , arising from the  $-7.28 \text{ MeV}$   $Q$ -value for the ground state decay. As this expression is symmetrical about its maximum at  $(1/3)E_{\text{max}}$ , the most probable value of  $E_\alpha$  and the mean value coincide at  $(1/3)E_{\text{max}}$ .

Considering only those alpha-particles having this most probable energy, and assuming an isotropic angular distribution in the carbon-12 centre of mass frame, a transformation to the laboratory frame of reference leads to an energy distribution whose maximum is at

$$E_{\alpha\text{lab}} = (E_{\text{max}} + T)/3,$$

where  $T$  is the recoil energy of the excited carbon-12 nucleus. It is therefore reasonable to suppose that the full energy distribution corresponding to each small  $T$  interval will be centred at the value given by this expression. (This is to be expected as a direct

result of the principle of conservation of energy and the assumption that the decay mechanism does not differentiate between the three alpha-particles).

With an accurate knowledge of the angular distribution of scattered neutrons leading to a particular energy level in the target nucleus, one could calculate the angular and hence energy distributions for the recoil. The latter would then determine the overall alpha-particle energy distribution. In the absence of such knowledge, the approximation used by Johnson (1963) has been adopted, in that the maximum and minimum values of  $T$  have been computed for each state, and the arithmetic mean taken as an estimate of the most probable recoil energy. This approximation gives the maximum of the single alpha-particle energy distribution at

$$E_{\text{lab}} = (E_{\text{ex}}(^{12}\text{C}) - 7.28 + \frac{1}{2}(T_{\text{min}} + T_{\text{max}}))/3$$

To arrive at the expected response of the anthracene crystal to three alpha-particles each having this energy, the non-linear response curve (figure 1.2) was used to determine the light output for one particle of this energy, the result obtained being then multiplied by a factor three.

Table 3.1 gives the results of this procedure for the 22 MeV case.

The locations of contributions predicted in this manner have been included in figure 3.4. One additional prediction has been

TABLE 3.1

Energies of particles arising from the  $^{12}\text{C}(n,n')^{12}\text{C}^*$  reaction at 22 MeV.

$^{12}\text{C}$ level	Scattered neutron energy		Total energy of 3 alphas			Most probable energy	Response	Effective energy
	min	max	max	min	mean	for each alpha	to (7)	for 3x (8)
(1)	(2)	(3)	(4)	(5)	(6)	(7)	(8)	(9)
7.65	9.30	14.27	5.42	0.45	2.94	0.98	74	2.5
9.638	7.67	12.23	7.05	2.49	4.77	1.59	128	3.8
10.1	7.38	11.77	7.34	2.95	5.15	1.72	141	4.1
10.844	6.70	10.99	8.02	3.73	5.88	1.96	165	4.6
11.828	5.90	9.97	8.82	4.75	6.79	2.26	200	5.3
12.713	5.20	9.04	9.52	5.67	7.60	2.53	230	5.8
13.352	4.69	8.37	10.03	6.35	8.19	2.73	253	6.3
14.083	4.11	7.59	10.61	7.13	8.86	2.96	280	6.7
14.710	3.63	6.92	11.09	7.80	9.45	3.15	305	7.2
15.109	3.32	6.50	11.40	8.22	9.81	3.27	320	7.4
16.106	2.56	5.42	12.16	9.30	10.73	3.58	360	8.1
16.677	2.21	4.90	12.51	9.82	11.17	3.72	375	8.3
17.23	1.73	4.17	12.99	10.55	11.77	3.92	410	8.8
17.77	1.34	3.56	13.38	11.16	12.27	4.09	420	9.0
18.36	0.94	2.88	13.78	11.84	12.81	4.27	440	9.3
18.39	0.92	2.84	13.80	11.88	12.84	4.28	440	9.3

(continued)

TABLE 3.1 (continued)

(1)	(2)	(3)	(4)	(5)	(6)	(7)	(8)	(9)
18.71	0.71	2.46	14.01	12.26	13.14	4.38	450	9.5
18.84	0.63	2.31	14.09	12.41	13.25	4.42	455	9.6
19.2	0.41	1.86	14.31	12.86	13.59	4.53	470	9.7
19.39	0.30	1.62	14.42	13.10	13.76	4.59	480	9.9
19.88	0.06	0.95	14.66	13.77	14.22	4.74	510	10.2
20.24	0.00	0.03	14.72	14.69	14.71	4.90	530	10.5

included, namely the expected location of the falling edge arising from the reaction  $^{12}\text{C}(n,\alpha)^9\text{Be}$ . (The contribution of the  $^9\text{Be}$  scintillation is neglected. This is justified by an extrapolation of the succession of curves in figure 1.2 to heavier ions). For the 22 MeV case, this energy has been calculated as 15.24 MeV; all three predictions are included in the figure, and are labelled E. These positions provide useful calibration points; a slight adjustment of the energy scale ~~is seen to be required.~~ **has been made.**

It is seen from the carbon-12 level density above the level at 9.63 MeV that contributions to the alpha spectra cannot be attributed to the decay of specific individual states. The states may be grouped, however, and regions of each spectrum associated with a group of states. Thus at 22 MeV bombarding energy, for example, two strong contributions are seen to be peaked at approximately 8 and 9 MeV effective alpha-particle energy, and it is reasonable to associate these with the states at 16.106 and 16.577 MeV, and with those from 17.23 to 18.71 MeV, respectively.

Such allocations are, of course, dependent on the validity of the assumption of symmetry for the three alpha-particles emitted in a disintegration, the assumption upon which the energies of Table 3.1 are based. The results of Frye, Rosen and Stewart (1955), obtained from analysis of three-pronged stars produced in a nuclear emulsion irradiated by fast neutrons, have shown that as the bombarding energy increases from 13 to 20 MeV, this assumption approaches the true situation more and more closely. In other words, for high bombarding energies, the role of an intermediate nucleus

${}^8\text{Be}$ ) becomes unimportant to the decay process.

A consideration of the small, positive Q-value for the decay  ${}^8\text{Be}(\text{g.s.}) \rightarrow 2\alpha$  (+0.09 MeV, see figure 1.1) shows that if a state in carbon-12 decays sequentially via the ground state of beryllium-8 to alpha-particles, nearly all the kinetic energy associated with the decay will be taken by the alpha-particle emitted first. The non-linear response of anthracene to alpha-particle energy (figure 1.2) ensures that the total light output of the phosphor will be greater in this case than if the energy were distributed equally among the three alpha-particles produced.

It is suggested that this phenomenon, due to a corresponding non-linear response for stilbene, can be used to explain why alpha-particle groups, detected by Johnson (1963) in pulse-height spectra derived from a stilbene crystal under 14.85 MeV neutron bombardment, were found at pulse-heights about 30% greater than those predicted from calculations equivalent to those of Table 3.1, but neglecting the effects of non-linear response. Had Johnson taken response curves for stilbene into account, the margin of apparent error would have been greater. Comparing the bombarding energy in the stilbene experiment with the 13 to 20 MeV range investigated by Frye et al, it indeed seems likely that the part played by the beryllium-8 nucleus accounts for the observed discrepancy.

In the present experiment it is seen that there is a trend towards population of more highly excited levels as the bombarding

energy is increased. Considered in conjunction with the results of Frye et al. discussed above, this suggests that it is the lower-lying states that decay sequentially via the beryllium-8 nucleus, and that for the disintegration of carbon-12 nuclei in states of high excitation, a single process is preferred.

The cross section for the reaction  $^{12}\text{C}(n,n')^{12}\text{C}^*(3\alpha)$ , for a broad range of states considered together, may be calculated from the spectra of figure 3.4, by comparison with the proton spectrum due to n-p scattering, the cross section for the latter process being well-known (Gammel, 1963). This has been done for the 22 MeV case. The theoretical curve of figure 3.5 was integrated over a 1 MeV range, and the fact that the proton energy spectrum is a rectangular distribution was used (see Appendix 1). The formula  $\text{C}_{14}\text{H}_{10}$  for anthracene was taken from Birks (1964). As the crystal was only a few mm. thick (and of non-uniform thickness), no attempt was made to correct for neutron flux attenuation.

It was assumed (Johnson, 1963) that the distribution of alpha-particle energies from the  $(n,\alpha)$  reaction was rectangular, and accordingly this contribution, extending from 6.02 to 15.24 MeV, was subtracted as a slowly rising function. The remaining number of events producing apparent alpha-particle energies above 6 MeV gave an estimate for the  $^{12}\text{C}(n,n')^{12}\text{C}^*(3\alpha)$  cross section proceeding via the states from 13.352 to 20.24 MeV excitation energy, of 34 mb. The cross section for the  $^{12}\text{C}(n,\alpha)^9\text{Be}$  reaction was obtained as 15 mb.

In calculating the latter value, the alpha-particle contribution

from 6.02 to 15.24 MeV referred to above was used. Thus the assumption was made that beryllium-9 nuclei were always formed in the ground state. This assumption is based on the findings of Frye et al. (1955) for 20 MeV neutrons, although Barjon (1962) reports evidence of excitations of this nucleus to a level at 6.76 MeV when carbon is irradiated with 15 MeV neutrons.

In the following section it is seen how alpha-particle contributions detected in anthracene under 22 MeV neutron bombardment may be associated with specific levels of the carbon-12 nucleus.

#### 3.4 THE COINCIDENCE EXPERIMENT

When the coincidence modification was first devised for the dual parameter system, the motive was to inhibit the storage of n-p scattering data. This modification possesses two major additional features which facilitate the recording of well-defined spectra due to alpha-disintegration of carbon-12 levels only.

The structure of the alpha spectra presented in the previous sections was masked by a broad background due to the  $(n,\alpha)$  reaction on carbon-12. Events in the anthracene crystal are now to be recorded only in coincidence with radiations scattered or emitted by the crystal; the recording of  $(n,\alpha)$  events leading to the ground state of beryllium-9 will thus be suppressed.

A close analysis of the spectra recorded without the coincidence circuitry was made difficult for a second reason. This was the considerable breadth of individual alpha-particle groups. This breadth

was contributed to by several factors.

Firstly, the non-linear response of anthracene led to the generation of different pulse-heights for different decay channels leading to the same final state. It has been shown in the previous section that this property could in a special case be applied to draw conclusions relating to the channel used. In the present case, however, the non-linearity is responsible for a loss of energy resolution, as a range of distributions of a given energy between three alpha-particles gives rise to a range of pulse-heights.

Secondly, the total kinetic energy of the three alpha-particles emitted in a disintegration of a carbon nucleus at a particular excitation energy, depended on the quantity of kinetic energy imparted to the nucleus by the primary inelastically-scattered neutron. This recoil energy depended on the recoil direction or, equivalently, on the neutron scattering angle.

A third factor introducing a spread of observed pulse-heights is the presence of the inelastically scattered neutron in the crystal. If this neutron undergoes a second collision in the crystal, a significant modification to the pulse height produced could occur. In view of the low detection efficiency of the crystal, however, (the crystal volume was only  $1.5 \text{ cm}^3$ ), this third factor may be expected to have the least serious effect of the three.

The coincidence system described fully in Chapter 2 virtually eliminated the effect of the second of these factors. Only those neutrons whose scattering angle lay in a range of approximately  $45^\circ$

about a mean value of approximately  $135^\circ$  reached the shielded detector of figure 2.4.

It is probable that the third reason for broadening of alpha peaks described above is affected by the coincidence requirement. For a thicker scatterer this aspect might become significant; it could perhaps be taken into account by using a Monte Carlo calculation approach.

The effects of the phosphor's non-linear response remain unaffected.

The system of Section 2.4 was assembled for the recording of dual parameter data arising from neutron-carbon interactions in the anthracene crystal, and calibrated according to the procedure outlined in Section 2.4A. Two sets of runs were carried out, one with 11 cm of the dense lead alloy described in Section 2.4 interposed between the target and the plastic phosphor, and one with twice this length of shield. The anthracene was positioned directly in front of the target and as close to it as possible within the limits allowed by the geometry in each case. The deuteron beam energy was set at 5 MeV, thus selecting a neutron energy of 22 MeV at the crystal.

Each set consisted of one run with the time-of-flight window (see Section 2.4A) selected to record true coincidences, one run for recording random coincidences only, and a short third run to record data with no coincidence requirement. The duration of the first two runs in each case was approximately three hours.

As the counts accumulated in the background runs were considerable

in each case, it was desirable to introduce some form of numerical smoothing to the data before subtraction of the two-parameter background spectra. The approach adopted for each 32 x 128 channel array was to smooth each 32-element PSD spectrum by the method of fourth differences (see Appendix 3). The method required the fitting of several thousand least-squares parabolas per run, and it was decided to write a computer program to carry out the numerical procedures involved (see program MDIFF, Appendix 4). Both the true coincidence and random coincidence runs were smoothed before subtraction. The number of counts recorded by the boron long counter for each run was used to normalise the random coincidence spectrum to the real coincidence spectrum in each case.

For the case where the greater length of shield was used to screen the plastic from the neutron source, it was found that the signal to background ratio in the region of interest was low. The result was that in the region of interest of the net matrix, obtained by subtraction after smoothing, the statistical errors were similar in magnitude to the respective counts. This was not so for the case in which the thinner shield was used. The interpretation is that there was a high flux of background radiation in the general vicinity of the neutron source, whose dependence on distance from the source was much less sensitive than inverse square dependence. This would arise from neutron scattering at the bulky liquid nitrogen tank and cold trap assembly, and at the ceiling, floor and walls of the neutron area (the nearest

wall was of concrete, 2 metres from the target) and from gamma ray production at the deuteron beam collimators and elsewhere.

It was found that due to a relatively low ratio of proton counts to alpha-particle counts in the matrix generated by the program MDIFF, the loci for these two groups were readily separable, without resorting to the techniques used in the computer program ALPHA, to an alpha-particle energy as low as 4 MeV. In figure 3.6, the pulse height distribution of counts recorded in the alpha-particle locus of this matrix is presented, for the data set recorded at the closer distance from the target. The errors given represent the sum of counting errors for the two runs.

The data accumulated with the coincidence switched off were used to provide the 15.24 MeV alpha-particle energy calibration point referred to in Section 3.3. The energy scale calibration calculated from a Co-60 spectrum measurement, was confirmed.

The positions marked above the experimental points in figure 3.6 refer to the expected positions of alpha groups. They have been calculated using the technique of Section 3.3, but for the range of n-C scattering angles appropriate to the geometry of the coincidence experiment. This angular range was taken to be  $135^\circ \pm 23^\circ$ . A shift in expected peak location of up to 1.4 MeV results from this restriction of the scattering angle. Table 3.2 shows how the results were obtained.

It appears from figure 3.6 that the carbon-12 states of excitation energy from 7.65 to 13.352 MeV all play a significant part in

TABLE 3.2

Energies of particles

arising from the  $^{12}\text{C}(n,n')^{12}\text{C}^*$  reaction at 22 MeV.

Neutron scattering angle  $135^\circ$

$^{12}\text{C}$ level	Scattered neutron energy	Total energy of 3 alphas	Most probable energy for each alpha	Response to (4)	Effective energy for 3x (5)
(1)	(2)	(3)	(4)	(5)	(6)
7.65	9.73	4.99	1.66	135	3.9
9.638	8.06	6.66	2.22	195	5.2
10.1	7.67	7.05	2.35	210	5.5
10.844	7.06	7.66	2.55	232	5.8
11.828	6.24	8.48	2.83	264	6.5
12.713	5.51	9.21	3.07	295	7.0
13.352	4.99	9.73	3.24	315	7.3
14.083	4.39	10.33	3.44	344	7.8
14.710	3.88	10.84	3.61	360	8.1
15.109	3.56	11.16	3.72	375	8.3
16.106	2.77	11.95	3.98	414	8.9
16.577	2.40	12.32	4.11	422	9.0
17.23	1.90	12.82	4.27	440	9.3
17.77	1.49	13.23	4.41	455	9.6
18.36	1.06	13.66	4.55	472	9.7
18.39	1.04	13.68	4.56	473	9.7
18.71	.81	13.91	4.64	490	10.0
18.84	.72	14.00	4.67	494	10.0
19.2	.48	14.24	4.75	511	10.2
19.39	.36	14.36	4.79	520	10.4
19.88	.08	14.64	4.88	540	10.7
20.24	-	-	-	-	-

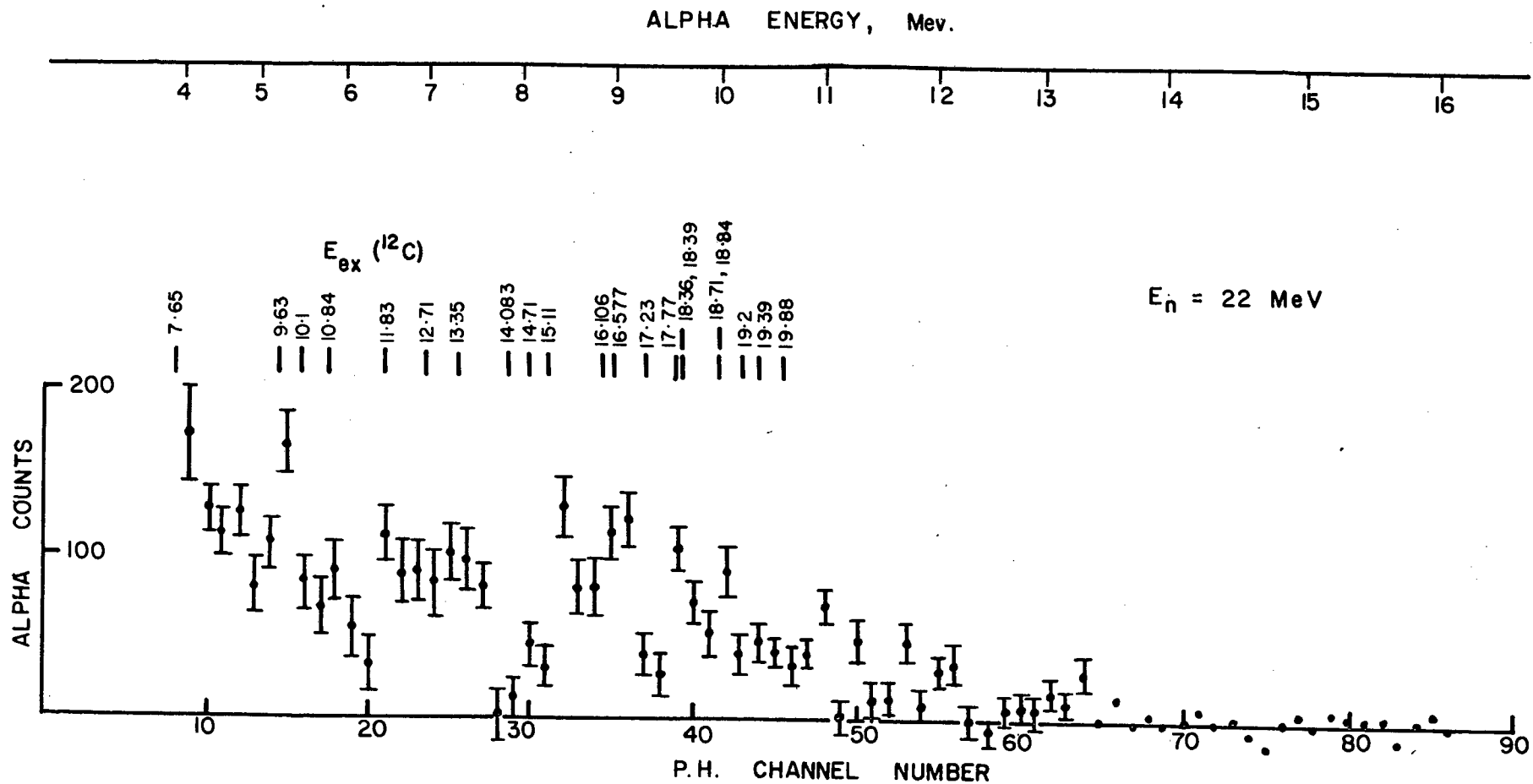


Fig. 3.6 Coincidence alpha spectrum in anthracene.

the reaction  $^{12}\text{C}(n,n')^3\alpha$  at 22 MeV. The presence of a broad band of alpha-particle energies corresponding to a level in the vicinity of 10.1 MeV excitation can not be isolated with certainty, although the single high count in channel 15 does appear to reside on a broad group extending from 6 MeV apparent alpha-particle energy down towards lower energies.

Many states of excitation energy greater than 13.352 MeV appear to contribute to the spectrum at higher energies. In particular the pair of levels at 16.106 and 16.577 MeV may be associated with the peak at channels 35 and 36. For still higher-lying levels it is difficult to make specific allocations; for levels of excitation greater than about 18.5 MeV, it is important to note that the scattered neutrons have energy near the detection threshold of the plastic phosphor, resulting in a loss of detection efficiency. The efficiency of this biased detector has not been investigated in detail (cf. Appendix 2), as the geometry calls for a highly sophisticated treatment. A Monte Carlo calculation approach similar to that of Batchelor et al. (1961) or Joseph et al. (1967) would provide an appropriate method.

As a check on the data smoothing and alpha-proton separation processes, the proton pulse-height spectrum of the net matrix has been examined. Marked structure is seen for proton energies below about 7 MeV (figure 3.7).

Much of this structure has been accounted for by consideration of the reaction  $^{12}\text{C}(n,n')^{12}\text{C}^*(p)^{11}\text{B}(g.s.)$ . The calculation of expected proton group locations, in terms of proton energy, is outlined below.

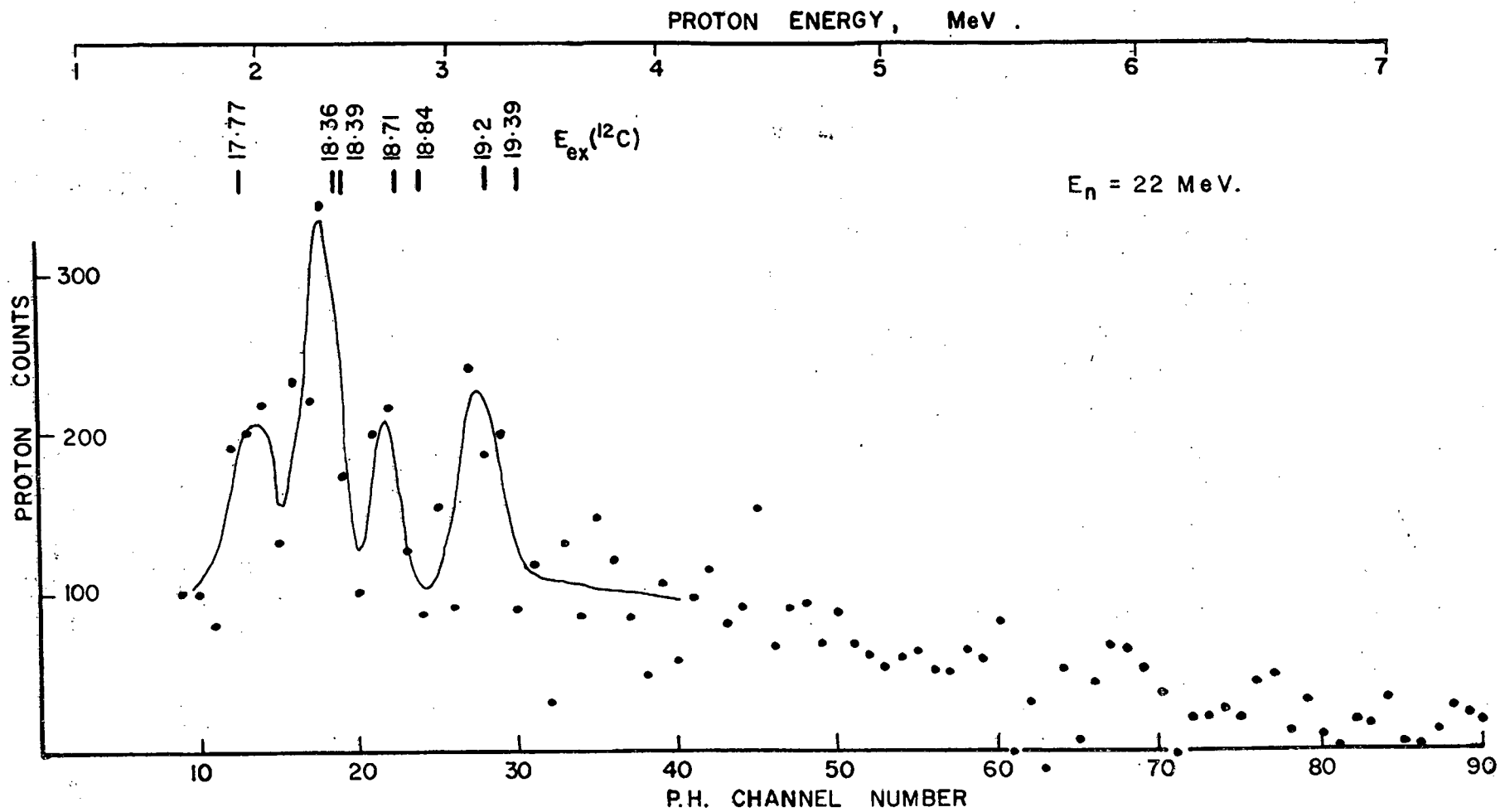


Fig. 3.7 Coincidence proton spectrum in anthracene.

previous section.

It must be pointed out that the results obtained with the system used in the coincidence experiment are subject to one source of ambiguity. Gamma rays produced by fast neutron interactions in the anthracene crystal, if detected in the plastic, can produce a "real" coincidence signal. Thus some alpha-particles from the reaction  $^{12}\text{C}(n,\alpha)^9\text{Be}^*(\gamma)^9\text{Be}$ , if it occurs (Frye et al. 1955, Barjon et al. 1962), could be included in the spectrum of figure 3.6. Or, if a decay such as  $^{12}\text{C}^*(\gamma,3\alpha)$  occurred, the coincidence signal corresponding to the process would be generated with exaggerated efficiency.

Two extensions of the system used in the present case could be employed to discriminate against such gamma rays. A longer scattered-neutron flight path could be used in conjunction with a time-of-flight discriminator; or pulse shape discrimination could be applied to the scattered-neutron detector. At an early stage of the present investigation, a system incorporating both of these features was designed and assembled; its full operation, however, required a triple parameter data storage system, to accumulate information relating to pulse height, pulse shape, and scattered-neutron time of flight for each scintillation in the scatterer fulfilling an appropriate fast coincidence requirement. A four parameter digital data storage system has since been constructed and is to be used in the three parameter experiment described above. Due to pressure of time, this experiment has had to fall outside the scope of this thesis.

CHAPTER 4.

MEASUREMENT OF NEUTRON  
SCATTERING CROSS SECTIONS

A unique interpretation of the alpha-particle spectra obtained by the methods of Chapter 3 was hampered by the close spacing of the carbon-12 levels involved. The aim of this neutron time-of-flight experiment was to attempt to determine angular distributions for inelastic scattering on individual highly excited states of the carbon-12 nucleus, leading to cross sections for the respective processes. It would appear (BNL 400, 1970) that no such data exist for neutron energies near 22 MeV. It was expected that this knowledge would define more closely the states whose alpha-decays were observed in previous experiments, and lead to an estimation of branching ratios for alpha-particle, proton, and hence also gamma ray decays of excited states of the nucleus of interest.

In point of fact the results were somewhat inconclusive. Section 4.3 is devoted to a discussion of why this was so, and of possible ways in which the experiment could be improved. A proposed multiparameter time-of-flight experiment is discussed. First of all, however, the experiment is described as it was performed.

#### 4.1 THE NEUTRON TIME OF FLIGHT EXPERIMENT

Using the system described in Sections 2.5 and 2.5A, time of flight spectra were recorded for monoenergetic neutrons scattered by carbon. Spectra were accumulated in pairs for five values of the laboratory scattering angle, ranging from  $37^\circ$  to  $142^\circ$ , one with the graphite cylinder in position, and one with it removed to record the neutron background, at each angle. The flight path was 2.47m in each case. Sets of ten spectra were recorded for two values of the

incident neutron energy, namely 22.00 and 20.05 MeV.

For the first set of spectra, the deuteron ion source used in the Van der Graaff accelerator was of the radio-frequency plasma-generation type, and the timing resolution was 3ns as already shown. At the time of lowering the deuteron energy for the second set of ten spectra, an Ortec duoplasmatron ion source was substituted for the RF source. The time scale was recalibrated from delayed prompt gamma ray peaks, recorded with the PSD switched off, and the timing resolution was now found to be 5ns. The effect of this deterioration was compensated for to some extent by the fact that flight times of slower neutrons were now being recorded. A recalibration of the time scale was necessary as slower deuterons were to traverse the finite distance between the inductive pulse pick-off and the tritium target.

A typical pair of spectra obtained from scattering of 22 MeV neutrons is shown in figure 4.1. The counts from channel number 400 to 500 were summed for each spectrum, and the entire background spectrum multiplied by the normalising factor required to make the 100-channel sums equal.

In spite of the very poor signal-to-background ratio reflected in figure 4.1, an attempt was made to subtract the background spectrum, to obtain the contribution of n-C scattering to the whole. It was essential to apply some form of numerical smoothing to the data before subtraction; on the other hand there is clearly a good deal of structure in the background spectrum which prohibits the use of "global" smoothing techniques. The local smoothing technique known as the

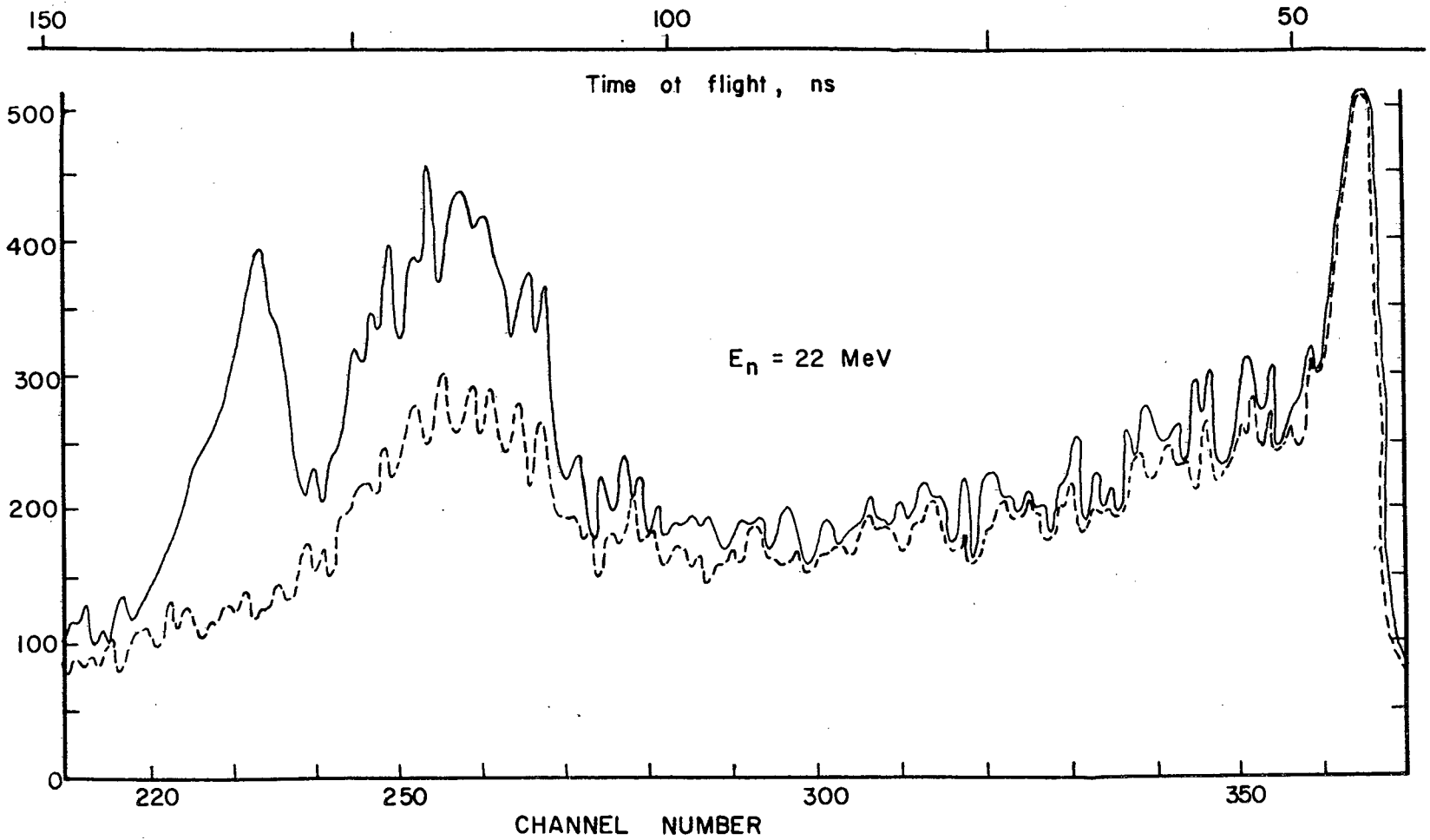


Fig. 4.1 Neutron time of flight spectra (----background) at  $142^\circ$ .

method of fourth differences (see Lanczos, 1957) was adopted, and is described in Appendix 3. Each spectrum of each pair was smoothed before subtraction. The difference between the spectra of figure 4.1 is shown in figure 4.2. Computer subroutines SMOOV and GRAPH (see Appendix 4.3) were written for the University's IBM 1130 computer to implement the smoothing and to give a visual presentation of the various spectra.

As a check on the possibility of the time scale altering due to drifts in the electronic circuits, each run was immediately followed by a short run with the PSD switched off, to record the position of the prompt gamma ray peak. In no case was the error determined in this fashion greater than one channel. The expected position of the peak due to neutrons proceeding directly from the tritium target through the shield to the neutron detector was also calculated for every spectrum. A peak was found within one channel of the expected position in every case. This result indicates that the neutron shield provided inadequate screening of the liquid scintillator.

#### 4.2 DISCUSSION OF RESULTS

A calculation was performed with the aid of the University's ICT 1301 computer to determine the expected positions on each of the time of flight spectra of neutron groups corresponding to the levels of carbon-12 that may be excited in the scattering sample. Calculated positions are shown for the spectrum of figure 4.2.

It is seen from the magnitude of the counting errors in this figure that no meaningful results for the number of counts corres-

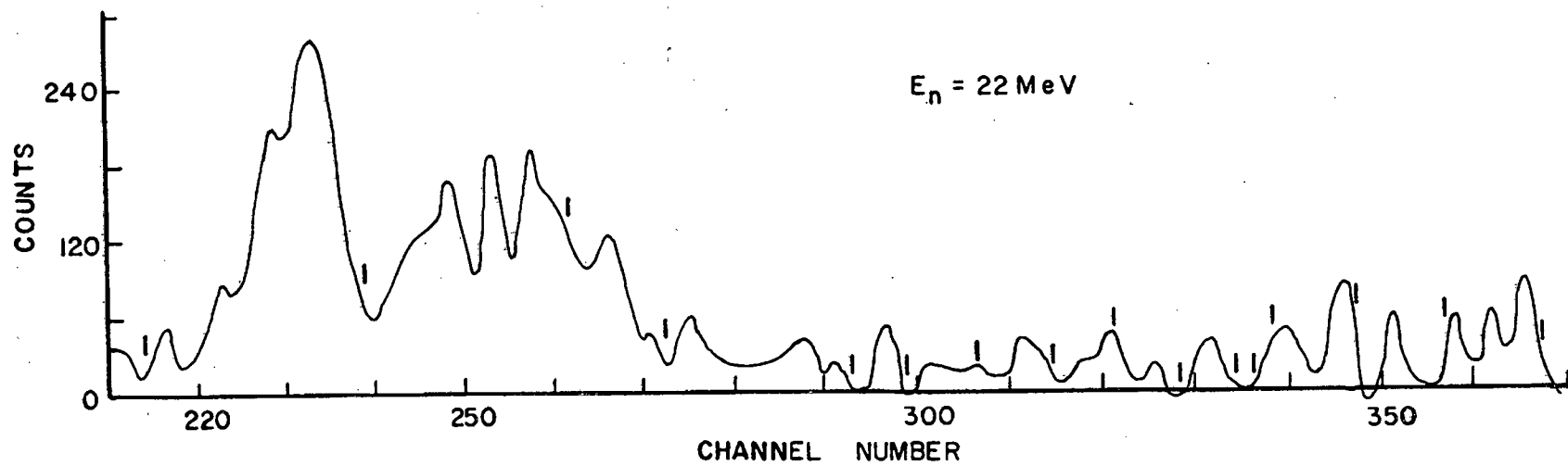


Fig. 4.2 Neutron time of flight spectrum at  $142^\circ$  after subtraction of background.

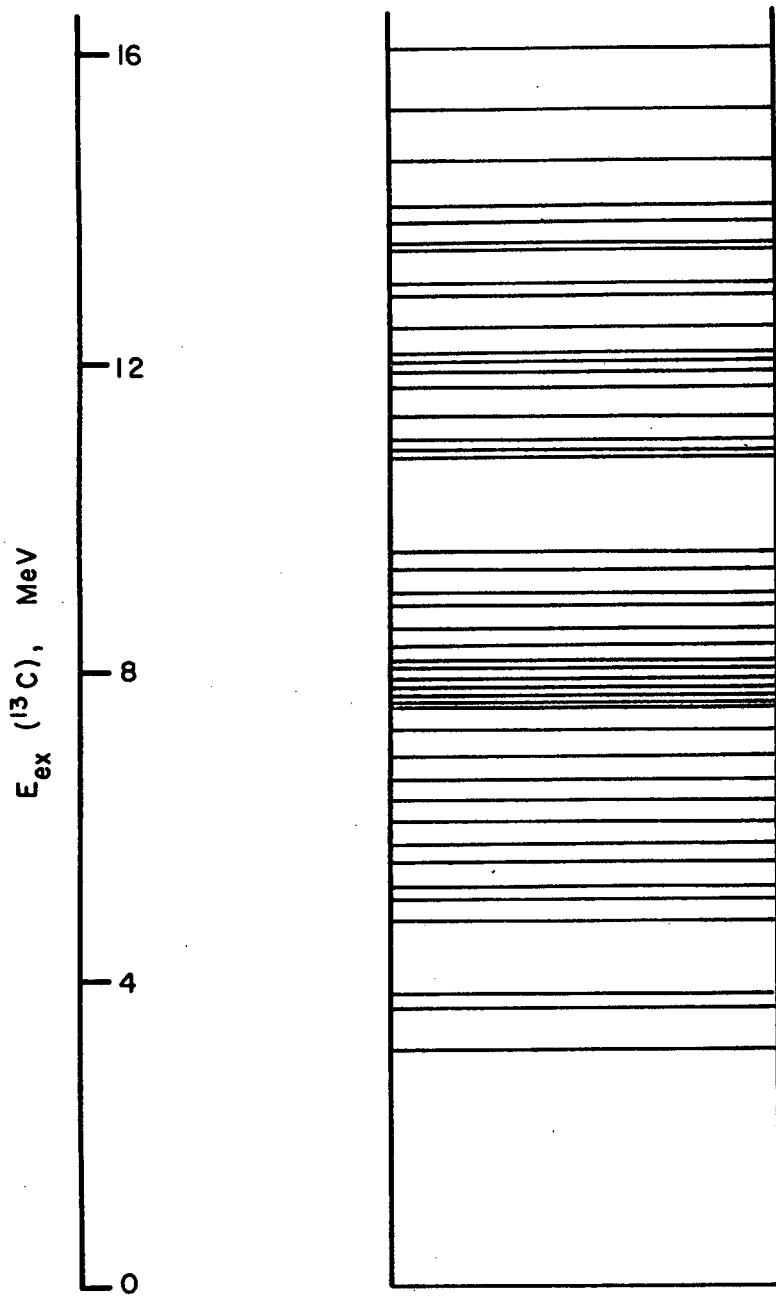


Fig. 4.3 Energy levels of Carbon - 13.

to provide a neutron flux able to compete with the background flux.

3. The shield was inadequate for screening fast neutrons. This is seen from the presence of a peak due to direct neutrons in every spectrum.
4. The neutron spectrum leaving the target vicinity was not entirely monochromatic, as seen from the run intended for permitting normalisation of data to absolute cross sections (Section 2.5A)
5. Too long a flight path was chosen in relation to the scatterer size; equivalently, the timing resolution (3ns) was too poor.

This was due to:

- i) The deuteron pulses being too broad.
- ii) As the fast anode pulse from the photomultiplier was required for pulse shape discrimination purposes, a slower dynode pulse was used to trigger the fast discriminator.
- iii) Discriminators operating on the amplitude of the leading edge of a pulse (as used here) cannot provide a high order of resolution in the presence of a broad spectrum of pulse heights.

Methods are in fact available for avoiding most of the above undesirable phenomena. Firstly, of course, the neutron scattering area should be large, and ideally of light construction.

A shield of intermediate screening properties is most undesirable. That used in the present experiment could be exchanged for a more

effective shield, or alternatively, if the neutron area were sufficiently large, an open geometry could be used.

The method of defining the starting time of a neutron's flight from the tritium target via the scatterer to the detector used here (i.e. inductive beam pick-off) has been improved upon by Barjon et al. (1962), Grin et al. (1965) and others, who have employed a counter situated in the accelerator vacuum to detect the production of alpha-particles in the  ${}^3\text{H}(d,n){}^4\text{He}$  reaction.

The timing resolution may be further improved by using a detector consisting of two photomultipliers viewing the same scintillator, one equipped with a PSD circuit, while the anode of the second tube is available for fast timing purposes. The tubes may be operated in coincidence and biased into the noise region to improve leading-edge discrimination (Batchelor et al. 1961) or better, a cross-over timing discriminator may be used instead.

A significant modification to neutron time of flight studies in the case of a carbon scatterer was introduced by Grin et al. (1965). The technique was to use two organic plastic scintillators, one as the neutron scatterer and the other to detect the scattered neutrons. Grin et al. estimated that as little as 10% of background radiation was recorded when this technique was used. This Lausanne group applied pulse shape discrimination to the neutron detector to eliminate gamma ray background, while amplitude discrimination was applied to the scatterer in order to select either those events producing low pulse-height carbon recoil scintillations, or those giving rise to alpha-

particles producing a somewhat higher range of pulse-heights. The bombarding energy was 14 MeV. Similar spectra were obtained for the two cases, except that the 9.63 MeV level was always seen to decay by alpha-particle emission.

As mentioned briefly at the close of Chapter 3, an experiment similar to that last mentioned is to be performed at S.U.N.I. The emphasis will be somewhat different, however. Pulse shape discrimination will be applied to the scatterer, to distinguish alpha-particle disintegrations from processes that give rise to protons. PSD applied to the second counter will serve to discriminate against gamma ray induced scintillations. Scattered neutron energies will be recorded by measurement of the true time of flight between the scintillators. The recording of three numbers per event, generated from the linear and gated PSD signals of the scatterer, and the neutron flight time, is envisaged. The magnetic tape multiparameter data storage system constructed at U.C.T. is to be used to accumulate the data. Successive replays of the magnetic tape should permit a two parameter pulse-height-PSD display to be generated for each excited state of carbon-12 populated, for level densities where timing resolution permits this.

On the basis of the excellent background suppression results of Grin et al. (1965, 1966, 1969), it is confidently hoped that the multiparameter neutron time-of-flight spectrometer to be used at 22 MeV will provide effective reduction of the background count rate; although background radiation effects may be expected to be more serious at the higher bombarding energy.

CHAPTER 5

SUMMARY

The pulse shape discrimination technique of particle identification in an organic phosphor has been effectively extended to distinguish reactions involving alpha-particles from other events in anthracene.

Resolution of particle type was found to be a sensitive function of the particle energies. Proton-gamma-ray resolution at a fraction of 1 MeV proton energy has been attainable for some time (Brooks, Pringle and Funt, 1960). The proton-alpha-particle resolution has been extended down to a proton energy of approximately 2 MeV, or 5 MeV apparent alpha-particle energy, by a numerical technique.

At the high neutron bombarding energies used, the accuracy of numerical methods for analysis of the low-energy data region was jeopardised by an excessive proton event count-rate. More than a general interpretation of the alpha spectra was further inhibited by recoil energy spread of decaying nuclei, and the presence of the  $(n,\alpha)$  background contribution.

These undesirable effects were annulled by the introduction of a second detector, alpha-particle and proton energy spectra being then readily obtained. Particle resolution was now unambiguously extended down to 1.5 MeV proton energy (4 MeV apparent alpha-particle energy).

The results of phase-space symmetry arguments, and kinematical considerations, combined with a knowledge of the anthracene light output as a function of particle type and energy, were

sufficient to attribute structure in the spectra obtained to particle decays of individual carbon-12 states proceeding via alpha-particle and proton emission respectively. Some indication has been found of a broad level near 10 MeV excitation energy.

For the structure of such spectra to be well defined, it is desirable that the second (coincidence) detector subtend a small range of the scattering angle at the scatterer. Effective background suppression, however, requires the solid angle subtended by this detector to be large. It is therefore suggested that the geometry of this experiment could be improved by substituting a symmetrical annular scintillator for the present cylindrical shape. For adequate light collection, the second detector should then comprise two photomultipliers. One of these could be used to provide pulse shape discrimination, to discriminate against possible contaminating reactions, and to reduce the background contribution still further. The anode signal from the remaining tube would then be available for fast coincidence purposes. The experience of the time-of-flight experiment suggests that an open geometry should be maintained, a single conical shield being used for screening from the neutron source. The annular scintillator could be a plastic, appropriately machined to facilitate a straightforward efficiency calculation.

The causes of an excessive background contribution have been discussed for a neutron time of flight experiment. A revised

approach has been suggested, incorporating a many-parameter analyser, of a type described in detail by Egelstaff and Rae (1964).

APPENDIX 1

PROTON RECOIL SPECTRUM

DUE TO

N - P SCATTERING IN ANTHRACENE

The differential cross section for n-p scattering may be described by the result of <sup>a</sup> partial wave analysis of the elastic scattering process. The scattering amplitude obtained is

$$f(\theta) = (1/k) \sum_{l=0}^{\infty} (2l+1) e^{i\delta_l} \sin \delta_l P_l(\cos \theta)$$

and the differential scattering cross section is given by

$$d\sigma/d\Omega_{cm} = |f(\theta)|^2 .$$

(See e.g. Messiah, 1961). Here  $k$  is the wave number of the incident neutron and  $l$  is its orbital angular momentum state, the  $\delta_l$  are the phase shifts experienced by the neutron waves due to the scattering potential, and  $P_l$  are Legendre polynomials whose argument is the cosine of the centre of mass scattering angle.

For n-p scattering at  $E_n = 22$  MeV, all the phase shifts are small except  $\delta_0$  and  $\delta_1$ ; and as interactions are much weaker in odd  $l$ -states than in even  $l$ -states (see e.g. Enge, 1966), we have almost pure s-wave scattering, leading to an angular distribution in the centre of mass frame which is nearly isotropic.

Angular distribution measurements at  $E_n = 22.5$  MeV, compiled by Wilson (1963) and presented in figure A1.1, support this. On the strength of this experimental result, an isotropic angular distribution in the centre of mass frame has been assumed in what follows.

Let us denote the total cross-section for s-wave scattering by  $\sigma_0$ . Then the differential scattering cross-section in the centre of

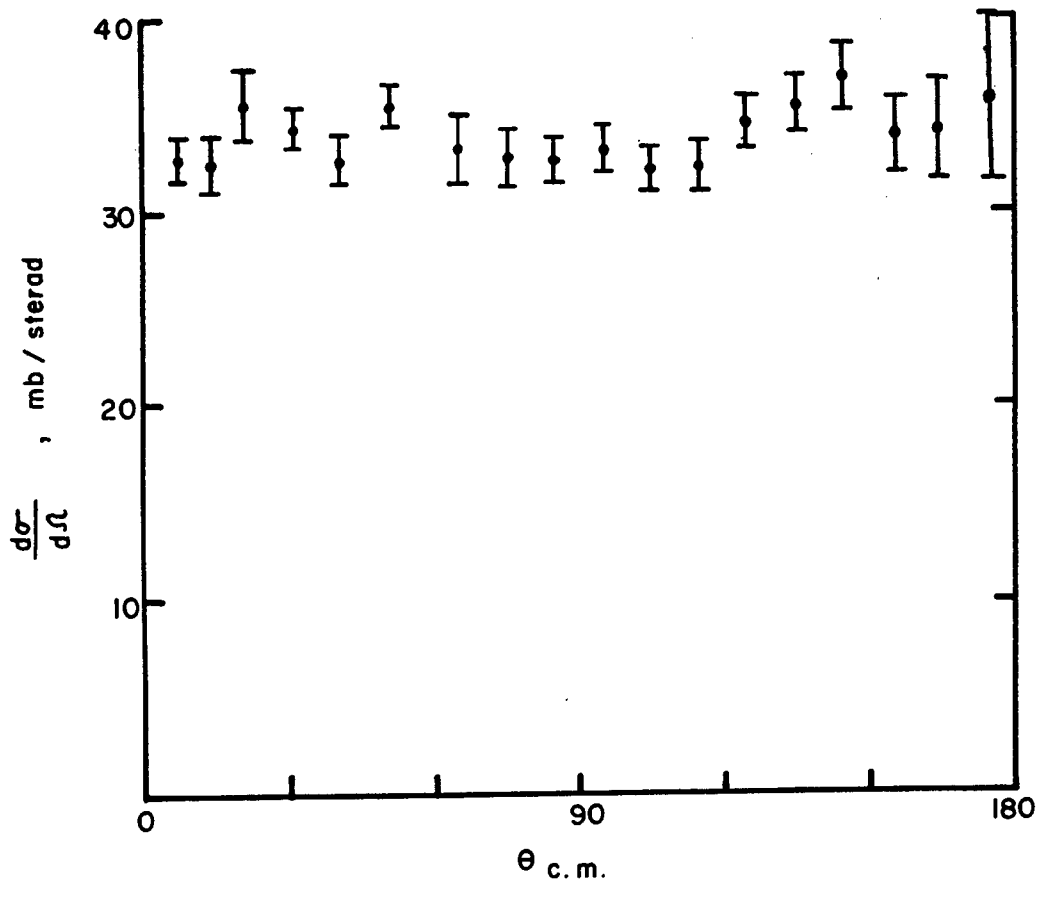


Fig.A.1.1 Experimental n-p scattering angular distribution results for  $E_n = 22.5$  MeV, compiled by Wilson (1963)

mass frame is

$$d\sigma_0/d\Omega = \sigma_0/4\pi,$$

and a (non-relativistic) transformation to laboratory coordinates yields

$$d\sigma_0/d\Omega_{lab} = (\sigma_0/\pi)\cos\theta_{lab} \quad (0 \leq \theta_{lab} \leq \pi/2) \quad \dots\dots(1)$$

for either the neutron or proton distribution. (The mass of a 22 MeV neutron differs from its rest mass by only 2.4%).

Taking  $\theta_{lab}$  as the angle of the proton recoil relative to the incident neutron direction, kinematical considerations show that the proton energy is given by

$$E_p = E_n \cos^2\theta_{lab} \quad \dots\dots(2)$$

where  $E_n$  is the incident neutron energy.

An examination of the reported response curves for anthracene (see figure 1.2) shows that the proton curve may be approximated by the expression

$$E_p = cL^b \quad \dots\dots(3)$$

over the energy range 1.5 to 7.5 MeV, which includes the proton energies encountered in the spectra of Chapter 3. Here L is the light output expressed in the units of figure 1.2, and the constants have the values

$$c = 0.0136, \quad b = 0.770 .$$

Assuming the graph to be precise, the maximum relative error incurred by this approximation is 4%.

Equations (2) and (3) together give the relation

$$\cos^2 \theta_{lab} = (1/E_n) c L^b$$

which on differentiation, becomes

$$2 \cos \theta_{lab} \sin \theta_{lab} d\theta_{lab} = (1/E_n) c b L^{b-1} dL. \quad \dots(4)$$

(Strictly, a negative sign should appear here because  $\cos \theta$  is a decreasing function while  $L^b$  is an increasing function. It is discarded). The axial symmetry of the system may be expressed by writing the element of solid angle at  $\theta_{lab}$  as

$$d\Omega_{lab} = 2\pi \sin \theta_{lab} d\theta_{lab} . \quad \dots(5)$$

Equations (1), (4) and (5) may be combined to show that the number of protons yielding light outputs between  $L$  and  $L + \Delta L$  is proportional to

$$\begin{aligned} & (d\sigma_0/d\Omega_{lab}) \cdot (d\Omega_{lab}/dL) \cdot \Delta L \\ & = (\sigma_0/E_n) c b L^{b-1} \Delta L . \quad \dots(6) \end{aligned}$$

For a linear photomultiplier and amplifier combination such as that used for measurement of alpha-particle spectra, the number of counts per pulse height channel is therefore proportional to (pulse height)<sup>b-1</sup>, where  $b-1 = -0.23$  for the energy range considered.

APPENDIX 2

NEUTRON DETECTION EFFICIENCY

OF A

BIASED SCINTILLATION COUNTER

The purpose of this section is to determine the neutron detection efficiency, as a function of neutron energy, of the 9.3 cm x 4.0 cm NE213 liquid scintillator cell used for the neutron time-of-flight experiment, and to correct the result for the effects of the externally imposed detection threshold (in this case the bias level of the fast discriminator).

If the scintillator, of thickness  $x$ , has  $n$  effectual scattering centres per unit volume, the uncorrected detection efficiency is given by the standard result (see e.g. Enge, 1966)

$$\text{efficiency} = 1 - e^{-n\sigma x} \quad \dots\dots(1)$$

where  $\sigma$  is the n-p scattering cross section (following Swartz and Owen (1960), who have shown that consideration of interactions with carbon makes a negligible difference to the efficiency). In the present case,  $x = 4\text{cm}$ . The number of protons per  $\text{cm}^3$  may be calculated from the composition and density of NE213. Batchelor et al. (1961) report that the density of this liquid is  $0.88\text{gm cm}^{-3}$ , and that the ratio of hydrogen atoms to carbon atoms is 1.21. Combining these quantities gives the result  $n = 4.85 \times 10^{22} \text{cm}^{-3}$ . Substitution of the values for  $n$  and  $x$  into equation (1) yields the equation

$$\text{efficiency} = 1 - e^{-0.194\sigma} \quad \dots\dots(2)$$

where  $\sigma$  is now expressed in barns. Values of  $\sigma$  for various energies have been taken from the tables of Gammel (1963), and are listed in

Table A2.1. Substitution of these values into equation (2) yields the uncorrected efficiency values in column 3 of the table.

The fast bias level calibration presented in figure 2.9 shows that the cut-off introduced by the bias is not sharp, but extends over about 20 channels. Smooth curves were drawn through the points of spectra (a) and (b) of figure 2.9 and used to calculate the efficiency correction introduced by the bias level at various channels. The result is shown in figure A2.1. The straight line drawn through the points approximates the data well, and it was accepted that the bias level introduces a correction factor which is zero below channel 9, unity above channel 26, and which varies linearly as the pulse-height excess over the channel-9 pulse-height between these values. The response curves for NE213 due to Batchelor et al. (1961) indicate that the light output depends linearly on electron energy for this scintillator, and approximately linearly on proton energy over not too large ranges. In what follows it is assumed that the dependence is linear.

Let us denote the proton energy at channel 9 by  $E_0$  and that at channel 26 by  $E_1$ , where the subscripts refer to the efficiency correction factors introduced by the bias level.

The assumption that NE213 has a linear response to proton energy is equivalent to assuming that a recorded recoil proton spectrum will be a rectangular distribution (seen by setting  $b = 1$  in equation (6) of Appendix 1). In this special case, the

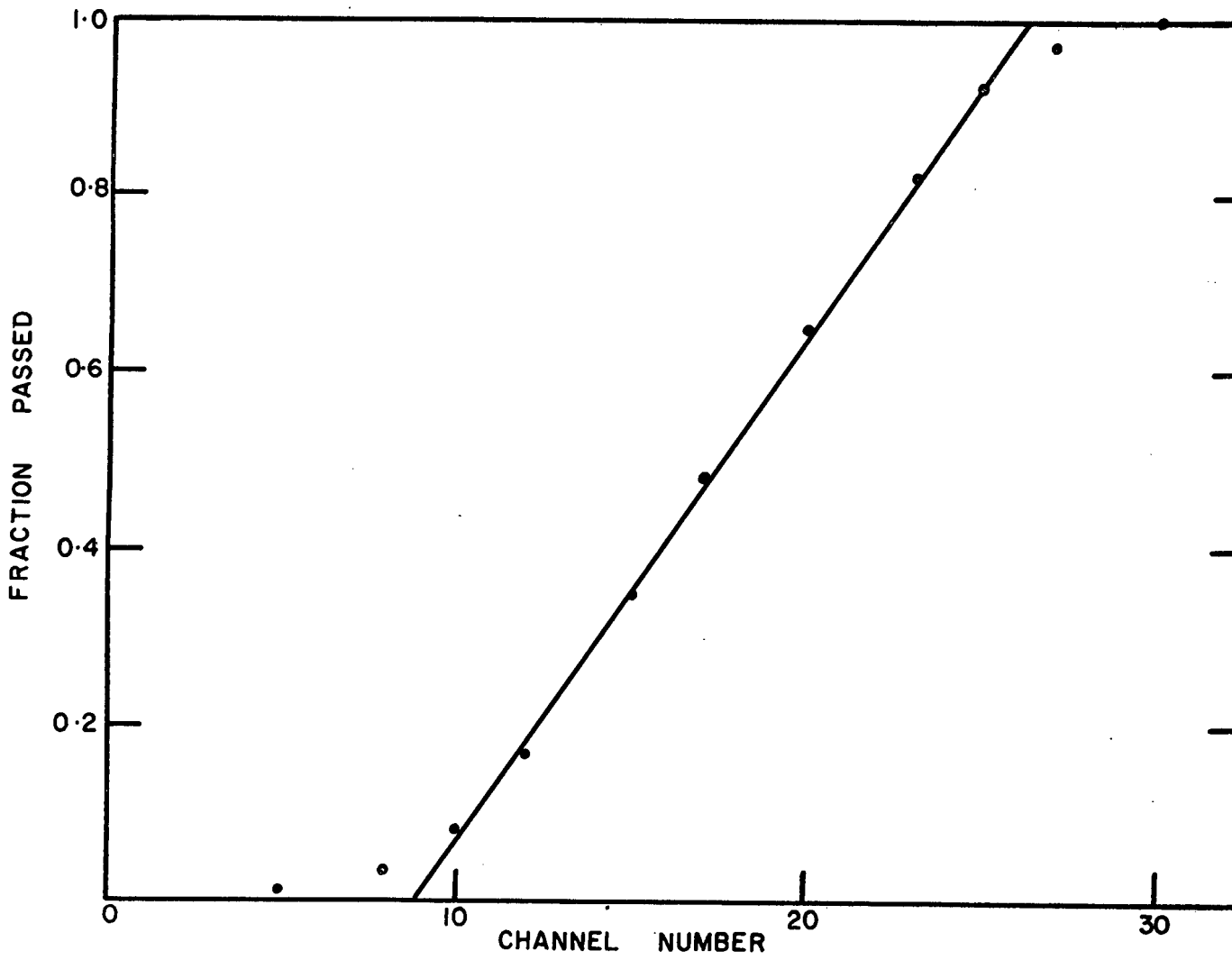


Fig. A2.1 Fraction of counts passed by fast discriminator.

TABLE A2.1

Neutron detection efficiency of NE213 cell.

<u>E</u> MeV	<u><math>\sigma_H</math></u> barn	<u><math>1 - e^{-n\sigma_H x}</math></u>	<u>F</u>	<u><math>\epsilon</math></u>
0.25	8.852	0.820	0.000	0.000
0.3	8.003	0.788	0.009	0.007
0.4	6.919	0.739	0.064	0.047
0.5	6.161	0.697	0.143	0.100
0.6	5.596	0.662	0.234	0.155
0.7	5.156	0.632	0.332	0.210
0.8	4.801	0.606	0.416	0.252
0.9	4.507	0.583	0.481	0.280
1.0	4.259	0.562	0.532	0.299
1.2	3.858	0.527	0.610	0.321
1.5	3.414	0.484	0.688	0.333
2.0	2.903	0.431	0.766	0.330
3.0	2.279	0.357	0.844	0.302
4.0	1.893	0.307	0.883	0.271
5.0	1.623	0.270	0.906	0.245
6.0	1.421	0.241	0.922	0.222
7.0	1.262	0.217	0.933	0.203
8.0	1.135	0.197	0.942	0.186
9.0	1.029	0.181	0.948	0.171
10.0	0.941	0.167	0.953	0.159
12.0	0.800	0.144	0.961	0.138
14.0	0.693	0.125	0.967	0.121
16.0	0.609	0.111	0.971	0.108
18.0	0.541	0.100	0.974	0.097
20.0	0.485	0.090	0.977	0.088
22.0	0.438	0.082	0.979	0.080

effect of the gradual cut-off on the detection efficiency may be evaluated as follows.

Suppose that in the absence of a detector bias level, a given number of monoenergetic neutrons of energy  $E$  incident at the detector give rise to a rectangular distribution having  $N$  counts per unit proton energy interval at each proton energy  $E_p$ , where  $E_p \leq E$ . Then the number of proton counts recorded is

$$\int_0^E N dE_p = NE. \quad \dots\dots(3)$$

We now introduce the bias level described above. For the case  $E \geq E_1$ , the number of recorded proton events is

$$\begin{aligned} & \int_{E_0}^{E_1} N (E_p - E_0)/(E_1 - E_0) dE_p + \int_{E_1}^E N dE_p \\ &= N(E - (E_0 + E_1)/2). \end{aligned} \quad \dots\dots(4)$$

Dividing the result (4) by (3), we obtain the detection efficiency correction factor

$$F = 1 - (E_0 + E_1)/2E \quad (E \geq E_1). \quad \dots\dots(5)$$

This is the standard result for a detector having a sharply-defined bias level at  $(E_0 + E_1)/2$  (see e.g. Owen, 1960).

This result does not apply, however, if the incident neutron energy lies below  $E_1$ . For this case where  $E$  lies between  $E_0$  and  $E_1$  we see that the number of recorded proton events is

$$\int_{E_0}^E N (E_p - E_0)/(E_1 - E_0) dE_p$$

$$= N(E - E_0)^2/2(E_1 - E_0). \quad \dots\dots(6)$$

Dividing this result by (3) yields the detection efficiency correction factor

$$F = (E - E_0)^2/(2E(E_1 - E_0)) \quad (E_0 < E < E_1) \quad \dots\dots(7)$$

For the third possibility, namely that E is less than E<sub>0</sub>, we have of course

$$F = 0 \quad (E \leq E_0) \quad \dots\dots(8)$$

Batchelor et al. (1961) have reduced their data for NE213 to the equations

$$L_e = E_e$$

$$L_p = 0.215E_p + 0.028 \quad (E_p \leq 8 \text{ MeV})$$

where the electron energy E<sub>e</sub> and the proton energy E<sub>p</sub> are in MeV, and L<sub>e</sub> and L<sub>p</sub> are the corresponding pulse-heights (normalised to electron energy units). For the fast bias calibration of figures 2.9 and A2.1, these equations show that

$$E_0 = 0.252 \text{ MeV}, \quad E_1 = 0.683 \text{ MeV}.$$

These values are used to calculate the efficiency correction F from equations (5), (7) and (8), and the results are given in column (4) of Table A2.1. The overall neutron detection efficiency ε is presented as a function of energy in column (5) of the Table, and also in figure A2.2.

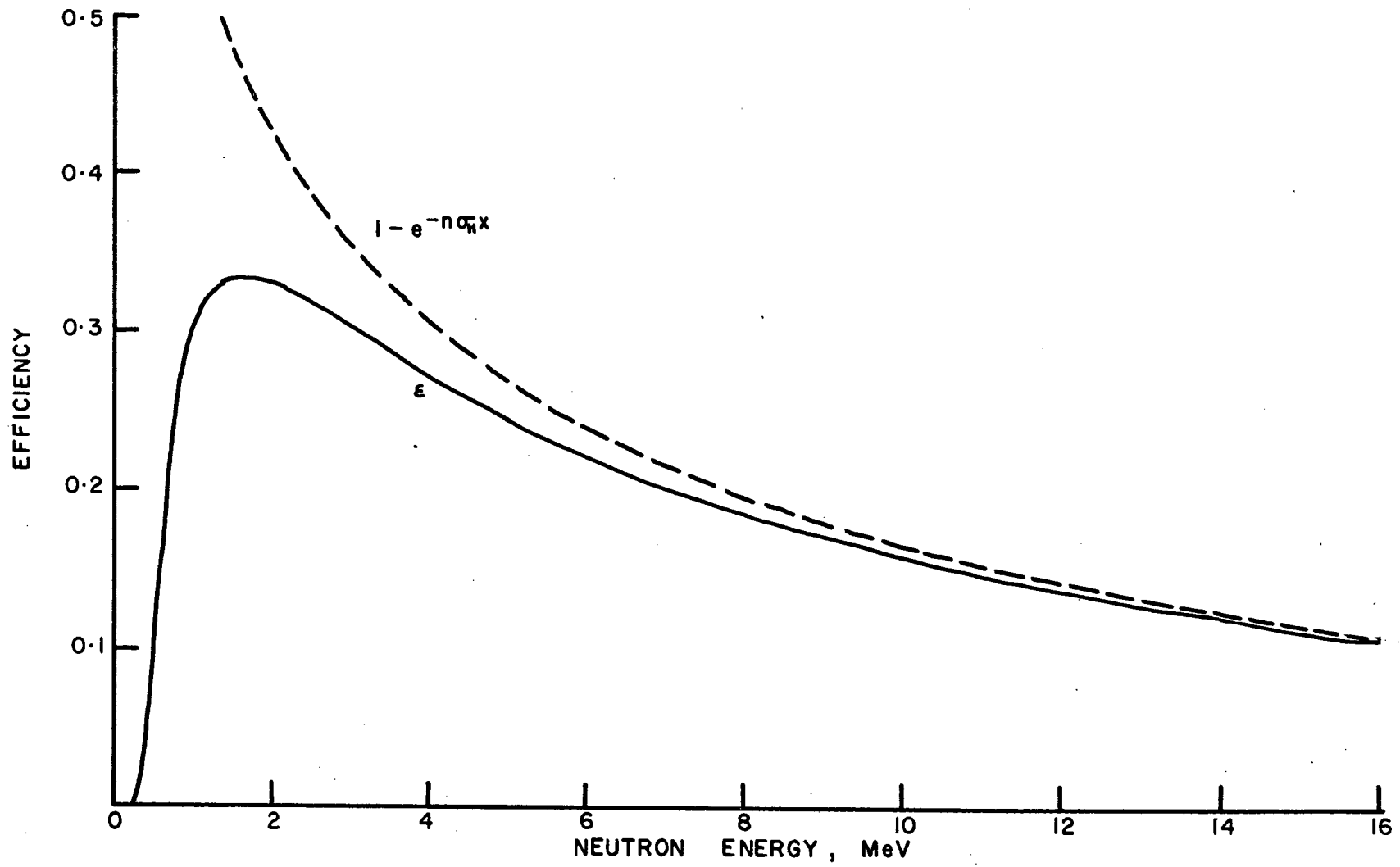


Fig. A2.2 Calculated neutron detection efficiency for NE213, 4cm. thick. Upper curve, zero bias. Lower curve, bias as used.

APPENDIX 3

DATA SMOOTHING

BY FOURTH DIFFERENCES

This local data smoothing technique is applicable to a spectrum of discrete data points  $(x_i, y(x_i))$  where the  $x_i$  are equally spaced (see Lanczos, 1957). The procedure for each given data point  $(x_k, y(x_k))$  is to fit the best parabola in the least squares sense to the five consecutive data points whose abscissae are  $x_{k-2}, x_{k-1}, x_k, x_{k+1}, x_{k+2}$ . The value of the parabola at  $x_k$  is regarded as the smoothed value of the centre point  $y(x_k)$ .

We wish to fit the parabola

$$y(x) = a_0 + a_1x + a_2x^2 \quad \dots\dots(1)$$

to the five points  $y(x_k)$  where  $x_k = -2, -1, 0, 1, 2$ . In order to minimise the mean square error...

$$E = \sum_k [a_0 + a_1x_k + a_2x_k^2 - y(x_k)]^2$$

we require

$$\frac{\partial E}{\partial a_i} = 0, \quad i = 0, 1, 2.$$

This set of equations reduces to

$$\begin{aligned} 5a_0 + 10a_2 &= \sum_k y(x_k) \\ 10a_1 &= \sum_k x_k y(x_k) \\ 10a_0 + 34a_2 &= \sum_k x_k^2 y(x_k). \end{aligned} \quad \dots\dots(2)$$

The ordinate of the parabola  $y(x)$  at  $x = 0$  is given simply by the constant  $a_0$ . We therefore solve the equations (2) for  $a_0$  to

obtain

$$a_0 = y(0) - (3/35)(y(-2) - 4y(-1) + 6y(0) - 4y(1) + y(2)) \dots\dots(3)$$

or, in terms of the central difference operator  $\delta$ ,

$$a_0 = y(0) - (3/35)\delta^4 y(0). \dots\dots(4)$$

Each value  $y(x_i)$  of the original spectrum may thus be replaced by the smoothed value

$$y(x_i) - (3/35)\delta^4 y(x_i)$$

except for the last two points at either end of the spectrum.

Equation (4) gives the technique its name. For computation, however, it is sometimes simpler to use equation (3) than to compile a table of differences to the fourth order. Equation (3) forms the basis for the computer subroutine SMOOV (see Appendix 4). It should be noted that the form of equation (3) does not permit any net gain or loss of counts ( $y$ ) in smoothing a large number of consecutive elements.

A modification of this technique makes it applicable to the special requirements of the analysis described in Chapter 3 for reduction of a two-parameter data matrix to component spectra.

It is required to determine the turning point of the least squares parabola described above. The turning point of the parabola of equation (1) is at

$$x_f = -(a_1/2a_2) \dots\dots(5)$$

Equations (2) are therefore solved completely to give

$$a_1 = (1/10)(-2y(-2) - y(-1) + y(1) + 2y(2)),$$

$$a_2 = (1/14)(2y(-2) - y(-1) - 2y(0) - y(1) + 2y(2)).$$

These expressions are substituted into equation (5). The result is that the least squares parabola fitted at five consecutive abscissae  $x_k = m-2, m-1, m, m+1, m+2$ , has its turning point at

$$x_t = m + x_f .$$

APPENDIX 4

COMPUTER PROGRAMS DEVELOPED  
FOR THE PRESENT INVESTIGATION

The three computer programs listed on the following pages were developed for the analysis of data recorded in experiments that are described in this thesis. These programs were written with the capabilities of the University's IBM 1130 computer in mind; all three have at some time also been run on an ICL 1903A computer.

The language used was Fortran IV, as modified for the 1130 machine. The flow of control for the first two programs, ALPHA and MDIFF, is fully described by the several "comment lines" interspersed among the Fortran statements. Such comments are distinguished by the appearance of the letter C in the first column, and are included purely for explanation. The third program, BACKS, is used mainly for data input and presentation, involving no major computation apart from data smoothing, already described in program MDIFF.

A4.1      PROGRAM ALPHA

C -- PROGRAM ALPHA	ALPHA
C -- GENERATES PROTON AND ALPHA SPECTRA FROM 2-PARAMETER PH-PSD DATA	ALPHA
C -- DEVELOPED FOR EXPERIMENTAL SERIES 4004 AT SUNI	ALPHA
DIMENSION TITLE(40), MAT(4100), PSD(32)	ALPHA 01
14 READ (2, 100) TITLE, NRUNS, FNORM	ALPHA 02
100 FORMAT (40A2 / 15, F10.0)	ALPHA 03
IF(NRUNS)99,99,10	ALPHA 04
10 WRITE (5, 200) TITLE, NRUNS, FNORM	ALPHA 05
200 FORMAT (1H1, 17X, 40A2 // 26X, 'NO. OF RUNS =', 15, '	ALPHA 06
1 NORMALISING FACTOR = ', F10.4 // 4X, 'PH-CHAN E-P-CHAN	ALPHA 07
2 P-PK-CHAN TOTAL ELECTRONS TOT-LESS-EL	PRALPHA 08
30TONS ALPHAS' /)	ALPHA 09
C -- READ IN MATRIX	ALPHA
DO 12 K= 1,4100	ALPHA 10
12 MAT(K) = 0	ALPHA 11
DO 60 I=1, NRUNS	ALPHA012
READ (2, 108) NFIRS, NLAST	ALPHA112
108 FORMAT (215)	ALPHA212
READ (2, 101) (MAT(K), K=NFIRS, NLAST)	ALPHA312
60 CONTINUE	ALPHA412
101 FORMAT (7X, I6, 9I7)	ALPHA 13
MIN = 1	ALPHA 14
DC 13 JCCL=2,128	ALPHA 15
C -- SET UP PSD SPECTRUM AT PH CHANNEL JCCL AND NORMALISE	ALPHA
DC 11 JROW=1,32	ALPHA 16
JADDR = 128*(JROW-1) + JCCL	ALPHA 17
PSD(JROW) = MAT(JADDR)	ALPHA 18
11 PSD(JROW) = FNORM * PSD(JROW)	ALPHA 19
C -- FIND CHANNEL OF LEAST COUNTS BETWEEN E AND P LCCI	ALPHA
C -- LIMIT CHANGE IN 'MIN' TO ONE PER UNIT CHANGE IN JCCL	ALPHA
IF (MIN - 2) 42,43,43	ALPHA 20
43 IF (PSD(MIN) - PSD(MIN-1)) 42,45,45	ALPHA 21
45 IF (PSD(MIN-1) - PSD(MIN+1)) 46,47,47	ALPHA 22
46 MIN = MIN - 1	ALPHA 23
GO TO 50	ALPHA 24
42 IF (PSD(MIN) - PSD(MIN+1)) 50,50,47	ALPHA 25
47 MIN = MIN + 1	ALPHA 26
50 KMIN = MIN	ALPHA 27

C -- FIND BEST FRACTIONAL VALUE OF MIN CHAN AND INTEGRATE TO THIS CHAN	ALPHA 27
C -- BUT FIRST ENSURE 'KMIN' IN RANGE 3 TO 30	ALPHA
IF (KMIN - 30) 15,15,16	ALPHA
15 IF (KMIN - 3) 18,17,17	ALPHA 28
16 KMIN = 30	ALPHA 29
GO TO 17	ALPHA 30
18 KMIN = 3	ALPHA 31
17 CALL MMINT(PSD,32,KMIN,CHMIN,SUMIN,INVAL)	ALPHA 32
GO TO (19,2,3,4,5),INVAL	ALPHA 33
C -- FIND CHAN OF MAX COUNTS AFTER 'MIN', IE PROTON PEAK CHANNEL	ALPHA 34
19 MIN1 = MIN + 1	ALPHA
MAX = MIN	ALPHA 35
DC 20 JROW = MIN1,32	ALPHA 36
IF (PSD(JROW) - PSD(MAX)) 20,20,21	ALPHA 37
21 MAX = JROW	ALPHA 38
20 CONTINUE	ALPHA 39
C -- FIND BEST FRACTIONAL VALUE OF MAX CHAN AND INTEGRATE TO THIS CHAN	ALPHA 40
C -- BUT FIRST ENSURE 'MAX' IN RANGE 3 TO 30	ALPHA
IF (MAX - 30) 25,25,26	ALPHA
25 IF (MAX - 3) 28,27,27	ALPHA 41
26 MAX = 30	ALPHA 42
GO TO 27	ALPHA 43
28 MAX = 3	ALPHA 44
27 CALL MMINT(PSD,32,MAX,CHMAX,SUMAX,INVAL)	ALPHA 45
GO TO (29,2,3,4,5),INVAL	ALPHA 46
C -- SUBTRACT INTEGRALS TO OBTAIN HALF PROTON PEAK AND CALCULATE ALPHA COUNTS	ALPHA 47
29 IF (JCOL - 50) 65,65,66	ALPHA 48
65 CRF = 2.0	ALPHA148
GO TO 67	ALPHA248
66 C50 = JCOL - 50	ALPHA348
CRF = 2.0 + (0.010286 * C50)	ALPHA448
67 PROTS = CRF * (SUMAX - SUMIN)	ALPHA548
TOT = 0.0	ALPHA 49
DO 30 JROW=1,32	ALPHA 50
30 TOT = TOT + PSD(JROW)	ALPHA 51
ALPHS = TOT - SUMIN - PROTS	ALPHA 52
C -- PRINT TOTAL, ELECTRONS, TOTAL MINUS ELECTRONS, PROTONS, ALPHAS	ALPHA
TMINE = TOT - SUMIN	ALPHA 53

WRITE (5, 202) JCOL, CHMIN, CHMAX, TOT, SUMIN, TMINE, PROTS, ALPHSALPHA	54
202 FORMAT (1H , 4X, I4, 2(8X,F6.2), 5(8X,F7.0))	ALPHA 55
GO TO 13	ALPHA 56
2 WRITE (5, 302) JCOL	ALPHA 57
302 FORMAT (1H , 4X, I4, ' INPUT TO MMINT OUTSIDE RANGE')	ALPHA 58
GO TO 13	ALPHA 59
3 WRITE (5, 303) JCOL	ALPHA 60
303 FORMAT (1H , 4X, I4, ' STRAIGHT LINE FITTED NOT PARABOLA')	ALPHA 61
GO TO 13	ALPHA 62
4 WRITE (5, 304) JCOL	ALPHA 63
304 FORMAT (1H , 4X, I4, ' TURNING POINT BELOW RANGE')	ALPHA 64
GO TO 13	ALPHA 65
5 WRITE (5, 305) JCOL	ALPHA 66
305 FORMAT (1H , 4X, I4, ' TURNING POINT ABOVE RANGE')	ALPHA 67
13 CONTINUE	ALPHA 68
C -- PLOT VISUAL REPRESENTATION OF DATA	ALPHA
70 G = 50.0	ALPHA168
CALL PHPSD(MAT,G,TITLE)	ALPHA268
C -- PREPARE FOR NEXT DATA SET	ALPHA
GO TO 14	ALPHA 69
99 CALL EXIT	ALPHA 70
END	ALPHA 71

SUBROUTINE MMINT(A, IDIM, J, CHAN, SUM, INVAL)	MMINT
DIMENSION A(32)	MMINT
INVAL = 1	MMINT
IF (J-IDIM+2) 20,20,2	MMINT
20 IF (J-3) 2,7,7	MMINT
C -- LEAST SQRS PARABOLA FITTED AT 5 CHANS CENTRE J AND TURNING PT FOUND	MMINT
7 TOP = 0.7*( A(J+1)-A(J-1) + 2.0*(A(J+2)-A(J-2)) )	MMINT
BOT = A(J+1)+A(J-1) - 2.0*(A(J+2)-A(J)+A(J-2))	MMINT
IF (ABS(BOT)-0.001) 3,8,8	MMINT
8 R = J	MMINT
CHAN = R + TOP/BOT	MMINT
IF (CHAN-0.5) 4,15,23	MMINT
23 INTPT = CHAN	MMINT
WHGLE = INTPT	MMINT
	.....

FRACT = CHAN - WHOLE	MMINT
POS = ABS(FRACT-0.5)	MMINT
IF (POS-0.001) 14,14,24	MMINT
24 CHAN = CHAN + 0.02959*SIN(6.2832*FRACT)/SQRT(POS)	MMINT
IF (CHAN-0.5) 4,15,14	MMINT
14 IF (CHAN-32.5) 15,5,5	MMINT
15 LAST = CHAN - 0.5	MMINT
R = LAST	MMINT
C -- INTEGRATE 'A' UP TO CHANNEL 'CHAN' INCLUDING FRACTIONAL PART	MMINT
SUM = 0.0	MMINT
IF (LAST) 4,12,13	MMINT
13 DO 10 K=1, LAST	MMINT
10 SUM = SUM + A(K)	MMINT
12 SUM = SUM + (CHAN-R-0.5)*A(LAST+1)	MMINT
RETURN	MMINT
2 INVAL = 2	MMINT
RETURN	MMINT
3 INVAL = 3	MMINT
RETURN	MMINT
4 INVAL = 4	MMINT
RETURN	MMINT
5 INVAL = 5	MMINT
RETURN	MMINT
END	MMINT
SUBROUTINE PHPSD(MAT1,GDIFF,TITLE)	PHPSD 01
C VISUAL PRESENTATION OF 2-PARAMETER DATA.	PHPSD
DIMENSION POINT(34), REC(115), MAT1(4100), TITLE(40)	PHPSD 02
DATA BLANK,AXIS,POINT/' ','I','2','3','4','5','6','7','8','9','A',	PHPSD 03
1 'B','C','D','E','G','H','J','K','L','M','N','P','Q','R','S',	PHPSD 04
2 'T','U','W','X','Y','Z','+','*','0','1'/	PHPSD 05
C CONVERT COUNTS-PER-STEP TO STEPS-PER-COUNT.	PHPSD
CONST = 1.0/GDIFF	PHPSD 06
C DIVIDE 128 ENERGY CHANNELS INTO 4 SECTIONS, ONE PLOTTED PER PAGE.	PHPSD
DO 55 JBLOC=1,4	PHPSD 07
J32CH = 32*JBLOC	PHPSD 08
JO1CH = J32CH-31	PHPSD 09

WRITE (5, 100) TITLE, JBLOC, J32CH, J01CH	PHPSD 10
100 FORMAT (1H1, 8X, 40A2, ' PAGE NUMBER', 12 /// 43X, 'ENERGY CH	PHPSD 11
CHANNELS', 15, ' TO', 15 //)	PHPSD 12
C SET UP ONE RECORD AT EACH OF THE 32 PSD CHANNELS.	PHPSD
DO 55 JROW = 1,32	PHPSD 13
C FILL RECORD WITH BLANKS AND PUT IN ONE AXIS ELEMENT PER ENERGY.	PHPSD
DO 56 J = 1,115	PHPSD 14
56 REC(J) = BLANK	PHPSD 15
DO 57 J = 1,32	PHPSD 16
JJ = 2*J + 1	PHPSD 17
57 REC(JJ) = AXIS	PHPSD 18
C LABEL PSD AXIS IF CURRENTLY AT 10, 20 OR 30.	PHPSD
IF(JROW-10) 60,61,60	PHPSD 19
61 REC(1) = POINT(34)	PHPSD 20
GO TO 66	PHPSD 21
60 IF(JROW-20) 62,63,62	PHPSD 22
63 REC(1) = POINT(1)	PHPSD 23
GO TO 66	PHPSD 24
62 IF(JROW-30) 64,65,64	PHPSD 25
65 REC(1) = POINT(2)	PHPSD 26
66 REC(2) = POINT(33)	PHPSD 27
C COMPUTE NO. OF STEPS FOR EACH ENERGY CHANNEL AT CURRENT PSD CHANNEL,	PHPSD
C ADD 0.5 FOR ROUNDING, LIMIT STEPS TO 50, OVERWRITE WHERE NECESSARY.	PHPSD
64 DO 50 JPH = 1,32	PHPSD 28
JCCL = 32*(JBLOC-1) + JPH	PHPSD 29
JADDR = 128*(JROW-1) + JCCL	PHPSD 30
COUNT = MAT1(JADDR)	PHPSD 31
NSTEP = CONST * COUNT + 0.5	PHPSD 32
IF(NSTEP-50) 51,51,50	PHPSD 33
51 J = 67 - 2*JPH + NSTEP	PHPSD 34
IF(J) 50,50,52	PHPSD 35
52 REC(J) = POINT(JPH)	PHPSD 36
50 CONTINUE	PHPSD 37
55 WRITE (5, 101) REC	PHPSD 38
101 FORMAT (1H , 115A1)	PHPSD 39
WRITE(5, 102)	PHPSD 40
102 FORMAT (1H1)	PHPSD 41
RETURN	PHPSD 42
END	PHPSD 43

A4.2      PROGRAM MDIFF

C -- PROGRAM	MDIFF	MDIFF
C -- NORMALISATION, SMOOTHING AND SUBTRACTION OF TWO 32X128 MATRICES	MDIFF	MDIFF
DIMENSION ARR(4100), JCOL(8), CUT(16), TITLE(20)	MDIFF	MDIFF 00
DEFINE FILE 1(4100, 2, U, KK)	MDIFF	MDIFF 01
C READ CONTROL CONSTANTS FOR FIRST MATRIX.	MDIFF	MDIFF
20 READ(2,101)TITLE,N1,FACT1,KPRIN,GDIFF	MDIFF	MDIFF 02
101 FORMAT(20A4/2(I5,F10.0))	MDIFF	MDIFF 03
IF (N1) 99,99,10	MDIFF	MDIFF 04
C READ, NORMALISE, SMOOTH FIRST MATRIX AND STORE ON DISKFILE.	MDIFF	MDIFF 05
10 CALL INSMO(ARR,N1,FACT1)	MDIFF	MDIFF 06
KK = 1	MDIFF	MDIFF 07
WRITE (1*KK) ARR	MDIFF	MDIFF 08
C READ CONTROL CONSTANTS FOR BACKGROUND MATRIX.	MDIFF	MDIFF 09
READ(2,102)N2,FACT2	MDIFF	MDIFF 10
102 FORMAT(I5,F10.0)	MDIFF	MDIFF 11
IF (N2) 99,99,19	MDIFF	MDIFF 12
C READ, NORMALISE, SMOOTH BACKGROUND MATRIX.	MDIFF	MDIFF
19 CALL INSMO(ARR,N2,FACT2)	MDIFF	MDIFF 13
C RECALL SMOOTHED MATRIX FROM DISKFILE + SUBTRACT SMOOTHED BACKGROUND.	MDIFF	MDIFF
DO 25 J=1,4100	MDIFF	MDIFF 14
KK = J	MDIFF	MDIFF 15
READ (1*KK) ELM	MDIFF	MDIFF 16
25 ARR(J) = ELM - ARR(J)	MDIFF	MDIFF 17
WRITE(5,103)TITLE,N1,FACT1,N2,FACT2,KPRIN,GDIFF	MDIFF	MDIFF 18
103 FORMAT(1H1,17X,20A4//12X,'NO. CHANNELS 1      FACTOR 1      NO. CH	MDIFF	MDIFF 19
1ANNELS 2      FACTOR 2      PRINT      DIFF. GRAPH' / 12X,	MDIFF	MDIFF 20
2      2(5X,14,7X,F10.3,8X),15,6X,F10.2	MDIFF	MDIFF 21
3      ///17X,'COUNTS AND	MDIFF	MDIFF 22
4 RUNNING TOTALS -- SUMMATION OVER ROW NUMBER FOR EACH COL	MDIFF	MDIFF 23
SUMN')	MDIFF	MDIFF 24
IF(KPRIN) 15,14,15	MDIFF	MDIFF 25
C FOR EACH ENERGY CHAN, PRINT NET COUNTS PER PSD CHAN, AND RUN-TOTS.	MDIFF	MDIFF
15 DO 7 JBLOC=1,16	MDIFF	MDIFF 26
DO 8 I=1,8	MDIFF	MDIFF 27
JCOL(I) = 8*(JBLOC-1) + I	MDIFF	MDIFF 28
8 OUT(2*I) = 0	MDIFF	MDIFF 029
WRITE(5,200)JCOL	MDIFF	MDIFF 30
200 FORMAT(1H0/1X,'COLUMN',6X,8( '(,I3,')',8X )/4X,'ROW')	MDIFF	MDIFF 31
DO 7 JROW=1,32	MDIFF	MDIFF 32

DO 7 JROW=1,32	MDIFF 32
DO 6 I=1,8	MDIFF 33
JADDR = (JROW-1)*128 + JCCL(I)	MDIFF 34
CUT(2*I-1) = ARR(JADDR)	MDIFF 35
6 OUT(2*I) = OUT(2*I) + CUT(2*I-1)	MDIFF 36
7 WRITE(5,201)JROW,( CUT(I),I=1,16)	MDIFF 37
201 FORMAT(1H ,2X,I4,1X,8(F6.0,F7.0))	MDIFF 38
14 IF(GDIFF) 20,20,5	MDIFF 39
C PLOT VISUAL REPRESENTATION OF NET MATRIX.	MDIFF
5 CALL PHP ( ARR,GDIFF,TITLE)	MDIFF 40
C PREPARE FOR NEXT DATA SET.	MDIFF
GO TO 20	MDIFF 41
99 CALL EXIT	MDIFF 42
END	MDIFF 43

SUBROUTINE INSMO(ARR,NCH,FNORM)	INSMO
DIMENSION ARR(4100), PSD(32), SMPSD(32)	INSMO
C SET PH PSD MATRIX, STORED LINEARLY, TO ZERO AND READ IN NCH POINTS.	INSMO
DO 10 J=1,4100	INSMO
10 ARR(J) = 0.0	INSMO
READ (2, 100) (ARR(J), J=1,NCH)	INSMO
100 FORMAT (7X, F6.0, 9F7.0)	INSMO
C MULTIPLY BY FNORM TO NORMALISE.	INSMO
DO 11 J=1,4100	INSMO
11 ARR(J) = ARR(J) * FNORM	INSMO
C FOR EACH OF THE 128 ENERGY CHANNELS,	INSMO
DO 12 JCOL=1,128	INSMO
C SET UP A 32 CHANNEL PSD SPECTRUM, SMOOTH, AND OVERWRITE.	INSMO
DO 13 JROW=1,32	INSMO
JADDR = 128*(JROW-1) + JCOL	INSMO
13 PSD(JROW) = ARR(JADDR)	INSMO
CALL SMOOV(32,PSD,SMPSD)	INSMO
DO 12 JROW=1,32	INSMO
JADDR = 128*(JROW-1) + JCOL	INSMO
12 ARR(JADDR) = SMPSD(JROW)	INSMO
RETURN	INSMO
END	INSMO

	SUBROUTINE SMOOV (N,RAW,SMOO)		SMOOV
	DIMENSION RAW( 32), SMOO( 32)		SMOOV
C	SMOOTH 'RAW' BY 4TH DIFFERENCES, EXCEPT 2 POINTS AT EACH END.		SMOOV
	SMOO(1) = RAW(1)		SMOOV
	SMOO(2) = RAW(2)		SMOOV
	SMOO(N-1)= RAW(N-1)		SMOOV
	SMOO(N) = RAW(N)		SMOOV
	K = N-2		SMOOV
	DO 40 I = 3,K		SMOOV
40	SMOO(I) = RAW(I) - 0.0857143*( RAW(I-2)+ RAW(I+2)		SMOOV
9	-4.0 *(RAW(I-1) + RAW(I+1) ) + 6.0 * RAW(I) )		SMOOV
	RETURN		SMOOV
	END		SMOOV

	SUBROUTINE PHP(ARR,GDIFF,TITLE)		PHP	01
C	VISUAL PRESENTATION OF 2-PARAMETER DATA.		PHP	
	DIMENSION POINT(34), REC(115), ARR(4100), TITLE(20)		PHP	02
	DATA BLANK,AXIS,POINT/' ','I','2','3','4','5','6','7','8','9','A',		PHP	03
1	'B','C','D','E','G','H','J','K','L','M','N','P','Q','R','S',		PHP	04
2	'T','U','W','X','Y','Z','+','*','0','1'/'		PHP	05
C	CONVERT COUNTS-PER-STEP TO STEPS-PER-COUNT.		PHP	
	CONST = 1.0/GDIFF		PHP	06
C	DIVIDE 128 ENERGY CHANNELS INTO 4 SECTIONS, ONE PLOTTED PER PAGE.		PHP	
	DO 55 JBLOC=1,4		PHP	07
	J32CH = 32*JBLOC		PHP	08
	JO1CH = J32CH-31		PHP	09
	WRITE (5, 100) TITLE, JBLOC, J32CH, JO1CH		PHP	10
100	FORMAT (1H1, 8X, 20A4, ' PAGE NUMBER', 12 /// 43X, 'ENERGY CH		PHP	11
	ANNELS', 15, ' TO', 15 ////)		PHP	12
C	SET UP ONE RECORD AT EACH OF THE 32 PSD CHANNELS.		PHP	
	DO 55 JROW = 1,32		PHP	13
C	FILL RECORD WITH BLANKS AND PUT IN ONE AXIS ELEMENT PER ENERGY.		PHP	
	DO 56 J = 1,115		PHP	14
56	REC(J) = BLANK		PHP	15
	DO 57 J = 1,32		PHP	16
	JJ = 2*J + 1		PHP	17
57	REC(JJ) = AXIS		PHP	18

C LABEL PSD AXIS IF CURRENTLY AT 10, 20 OR 30.	PHP	
IF(JRCW-10) 60,61,60	PHP	19
61 REC(1) = POINT(34)	PHP	20
GO TO 66	PHP	21
60 IF(JRCW-20) 62,63,62	PHP	22
63 REC(1) = POINT(1)	PHP	23
GO TO 66	PHP	24
62 IF(JRCW-30) 64,65,64	PHP	25
65 REC(1) = POINT(2)	PHP	26
66 REC(2) = POINT(33)	PHP	27
C COMPUTE NO. OF STEPS FOR EACH ENERGY CHANNEL AT CURRENT PSD CHANNEL,	PHP	
C ADD 0.5 FOR ROUNDING, LIMIT STEPS TO 50, OVERWRITE WHERE NECESSARY.	PHP	
64 DO 50 JPH = 1,32	PHP	28
JCOL = 32*(JBLOC-1) + JPH	PHP	29
JADDR = 128*(JROW-1) + JCOL	PHP	30
NSTEP = CONST * ARR(JADDR) + 0.5	PHP	31
IF(NSTEP-50) 51,51,50	PHP	32
51 J = 67 - 2*JPH + NSTEP	PHP	33
IF(J) 50,50,52	PHP	34
52 REC(J) = POINT(JPH)	PHP	35
50 CONTINUE	PHP	36
55 WRITE (5, 101) REC	PHP	37
101 FORMAT (1H , 115A1)	PHP	38
WRITE(5, 102)	PHP	39
102 FORMAT (1H1)	PHP	40
RETURN	PHP	41
END	PHP	42

A4.3      PROGRAM BACKS

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C -- PROGRAM BACKS
C -- BACKGROUND SMOOTHING BY FOURTH DIFFERENCES
C -- ARRAYS STORED 1 TO 512 BUT OUTPUT IS 0 TO 511 OR EQUIV
   DIMENSION SPECT(520),BACKG(520),SMSPE(520),SMBAC(520),SUBTR(520),
   1 RTOT(5),TITLE(40),L(5)
69 READ (2,103) TITLE
103 FORMAT (40A2)
   READ (2,100) NRUNS, KPRIN, DIFFG
100 FORMAT (2I5, F10.0)
   IF (NRUNS) 99,99,15
15 READ (2,101) NSPEC, FSPEC, KSMSP, RAWSG, SMOSG
101 FORMAT ( 2(I5, F10.0), F10.0)
   READ (2,102) (SPECT(K), K=1,NSPEC)
102 FORMAT (7X, F6.0, 9F7.0)
   DO 11 I=2,NSPEC
11 SPECT(I) = FSPEC * SPECT(I)
   IF ( KSMSP ) 13, 12, 13
12 IF ( NRUNS - 1 ) 99, 28, 45
45 DO 16 I=2,NSPEC
16 SMSPE(I) = SPECT(I)
   GO TO 14
13 CALL SMOOT (NSPEC, SPECT, SMSPE)
14 IF ( NRUNS - 1 ) 99, 28, 25
28 NBACK = 0
   KSMBA = 0
   FBACK = 0.0
   RAWBG = 0.0
   SMOBG = 0.0
   DO 1 I=1,NSPEC
   BACKG(I) = 0.0
1 SUBTR(I) = 0.0
   GO TO 29
25 READ (2, 101) NBACK, FBACK, KSMBA, RAWBG, SMOBG
   READ (2, 102) (BACKG(K), K=1,NBACK)
   DO 21 I=2,NBACK
21 BACKG(I) = FBACK * BACKG(I)
   IF (KSMBA) 23, 22, 23
22 DO 26 I=2,NBACK
26 SMBAC(I) = BACKG(I)
   GO TO 27
23 CALL SMOOT (NBACK, BACKG, SMBAC)
27 DO 30 I=2,NSPEC
30 SUBTR(I) = SMSPE(I) - SMBAC(I)
29 WRITE (5, 200) TITLE
200 FORMAT ( 1H1, 35X, 'BACKGROUND SMOOTHING AND SUBTRACTION'/////
   9 15X, 40A2 )
   WRITE (5, 201) NRUNS, KPRIN, DIFFG, NSPEC, FSPEC, KSMSP, RAWSG, SMOSG
201 FORMAT ( 1H0//66X, 'S P E C T R U M' / 20X, 'DIFFERENCE', 10X, '---
   9 -----'BACK49DS
8/ ' NRUNS PRINT SCALE CHANNELS NORM FACTORBACK50DS
7 SMOOTHING RAW SCALE SMOOTHED SCALE' / 15, 4X, 15, 6X, BACK51DS

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6 F10.2, 11X, 15, 6X, F10.4, 5X, 15, 7X, F10.2, 6X, 'F10.2')
WRITE (5, 202) NBACK, FBACK, KSMBA, RAWBG, SMOBG
202 FORMAT (1H0/64X, 'B A C K G R O U N D' / 40X, '-----')
9
8 ' CHANNELS NORM FACTOR
7 SMOOTHING RAW SCALE SMOOTHED SCALE' / 41X,
6 15, 6X, F10.4, 5X, 15, 7X, F10.2, 6X, F10.2)
WRITE (5, 203) SPECT(1), BACKG(1)
203 FORMAT(1H0, 'SPECTRUM -- CHANNEL ZERO = ', F7.0/
9 ' BACKGROUND -- CHANNEL ZERO = ', F7.0)
IF (KPRIN) 54, 60, 54
54 IF (KSMSP) 70, 71, 70
71 DO 72 I=2, NSPEC
72 SMSPE(I) = 0.0
70 IF (KSMBA) 59, 74, 59
74 DO 75 I=2, NBACK
75 SMBAC(I) = 0.0
59 WRITE (5, 204)
204 FORMAT (1H0///20X, 'S P E C T R U M', 26X, 'B A C K G R O U N D' /
9 ' CHAN', 8X, '-----', 8X,
8 '-----', 10X,
7 'NET RUN' / ' NO EXPTAL R-TOT SMOOTHED R-TOT
6 ' EXPTAL R-TOT SMOOTHED R-TOT
5 REALS TOT')
WRITE (5, 209)
209 FORMAT (1H, '-----'
3 '-----'
2 '-----' /)
DO 55 J= 1,5
55 RTOT(J) = 0.0
DO 56 I=2, NSPEC
RTOT(1) = RTOT(1) + SPECT(I)
RTOT(2) = RTOT(2) + SMSPE(I)
RTOT(3) = RTOT(3) + BACKG(I)
RTOT(4) = RTOT(4) + SMBAC(I)
RTOT(5) = RTOT(5) + SUBTR(I)
L(1) = SPECT(I) + 0.5
L(2) = SMSPE(I) + 0.5
L(3) = BACKG(I) + 0.5
L(4) = SMBAC(I) + 0.5
L(5) = SUBTR(I) + 0.5
II = I-1
56 WRITE (5, 205) II, L(1), RTOT(1), L(2), RTOT(2), L(3), RTOT(3),
1 L(4), RTOT(4), L(5), RTOT(5)
205 FORMAT (1H, 14, 2(8X, 16, F7.0, 7X, 16, F8.0), 8X, 16, F7.0)
60 IF ( RAWSG ) 61, 61, 62
62 WRITE (5, 401) TITLE, RAWSG
401 FORMAT(1H1, 15X, 40A2, //25X, 'GRAPH OF RAW SPECTRUM WITH ',
9 F10.2, ' COUNTS PER STEP')
CALL GRAPH(NSPEC, RAWSG, SPECT)
61 IF ( SMOGS ) 63, 63, 64

```

```

BACK52DS
BACK53DS
BACK54DS
BACK55DS
BACK56DS
BACK57DS
BACK58DS
BACK59DS
BACK60DS
BACK61DS
BACK62DS
BACK63DS
BACK64DS
BACK65DS
BACK66DS
BACK67DS
BACK68DS
BACK69DS
BACK70DS
BACK71DS
BACK72DS
BACK73DS
BACK74DS
BACK75DS
BACK76DS
BACK77DS
BACK78DS
BACK79DS
BACK80DS
BACK81DS
BACK82DS
BACK83DS
BACK84DS
BACK85DS
BACK86DS
BACK87DS
BACK88DS
BACK89DS
BACK90DS
BACK91DS
BACK92DS
BACK93DS
BACK94DS
BACK95DS
BACK96DS
BACK97DS
BACK98DS
BACK99DS
BACKA0DS
BACKA1DS
BACKA2DS

```

```

64 WRITE (5, 402) TITLE, SMOSG
402 FORMAT(1H1, 15X, 40A2, //, 22X, 'GRAPH OF SMOOTHED SPECTRUM',
9 ' WITH ', F10.2, ' COUNTS PER STEP')
CALL GRAPH(NSPEC, SMOSG, SMSPE)
63 IF ( RAWBG ) 65, 65, 66
66 WRITE (5, 403) TITLE, RAWBG
403 FORMAT (1H1, 15X, 40A2, //, 24X, 'GRAPH OF RAW BACKGROUND WITH
9', F10.2, ' COUNTS PER STEP')
CALL GRAPH(NBACK, RAWBG, BACKG)
65 IF ( SMOBG ) 67, 67, 68
68 WRITE (5, 404) TITLE, SMOBG
404 FORMAT (1H1, 15X, 40A2, //, 22X, 'GRAPH OF SMOOTHED BACKGROUND
9ITH ', F10.2, ' COUNTS PER STEP')
CALL GRAPH(NBACK, SMOBG, SMBAC)
67 IF ( DIFFG ) 69, 69, 79
79 WRITE (5, 405) TITLE, DIFFG
405 FORMAT (1H1, 15X, 40A2, //, 5X, 'GRAPH OF SPECTRUM MINUS BACKG
9ROUND WITH SMOOTHING AS REQUESTED. SCALE ', F10.2, ' COUNTS
8 PER STEP')
CALL GRAPH(NSPEC, DIFFG, SUBTR)
GO TO 69
99 CALL EXIT
END
SUBROUTINE SMOOT (N, RAW, SMOO)
DIMENSION RAW(520), SMOO(520)
SMOO(1) = RAW(1)
SMOO(2) = RAW(2)
SMOO(3) = RAW(3)
SMOO(N-1) = RAW(N-1)
SMOO(N) = RAW(N)
K = N-2
DO 40 I = 4, K
40 SMOO(I) = RAW(I) - 0.0857143*( RAW(I-2)+ RAW(I+2)
9 -4.0 *(RAW(I-1) + RAW(I+1) ) + 6.0 * RAW(I) )
RETURN
END
SUBROUTINE GRAPH(N, SCALE, POINT)
REAL LINE(111)
DIMENSION POINT(520)
DATA BLANK, AXIS, DOT/' ', 'I', 'X' /
DO 80 J=1, 111
80 LINE(J)=BLANK
CONST = 1.0/SCALE
WRITE (5, 302)
302 FORMAT(1H0//3X, '-10', 9X,
1 '0', 8X, '10', 8X, '20', 8X, '30', 8X, '40', 8X, '50', 8X,
2 '60', 8X, '70', 8X, '80', 8X, '90', 7X, '100')
LAB10 = 0
DO 12 LOOP=1, N
LABEL = LOOP-1
DO 6 I=1, 12

```

```

BACKA2DS
BACKA3DS
BACKA4DS
BACKA5DS
BACKA6DS
BACKA7DS
BACKA8DS
BACKA9DS
BACKB0DS
BACKB1DS
BACKB2DS
BACKB3DS
BACKB4DS
BACKB5DS
BACKB6DS
BACKB7DS
BACKB8DS
BACKB9DS
BACKC0DS
BACKC1DS
BACKC2DS
BACKC3DS
BACKC4DS
BACKC5DS
BACKS1DS
BACKS2DS
BACKS3DS
BACKS4DS
BACKS5DS
BACKS6DS
BACKS7DS
BACKS8DS
BACKS9DS
BACKT0DS
BACKT1DS
BACKT2DS
BACKT3DS
BACKG1DS
BACKG2DS
BACKG3DS
BACKG4DS
BACKG5DS
BACKG6DS
BACKG7DS
BACKG8DS
BACKG9DS
BACKH0DS
BACKH1DS
BACKH2DS
BACKH3DS
BACKH4DS
BACKH5DS

```

```

DATA BLANK, AXIS, DOT/ ' ', 'I', 'X'/
DO 80 J=1,111
80 LINE(J)=BLANK
CONST = 1.0/SCALE
WRITE (5, 302)
302 FORMAT(1H0//3X, '-10', 9X,
1 '0', 8X, '10', 8X, '20', 8X, '30', 8X, '40', 8X, '50', 8X,
2 '60', 8X, '70', 8X, '80', 8X, '90', 7X, '100')
LAB10 = 0
DO 12 LOOP=1,N
LABEL = LOOP-1
DO 6 I=1,12
J = 10*I-9
6 LINE(J) = AXIS
J = CONST * POINT(LOOP ) +11.5
IF (J-1) 2, 3, 3
2 J=1
3 IF (J-111) 4, 4, 5
5 J = 111
4 LINE(J) = DOT
IF (LABEL - LAB10) 10, 11, 10
11 WRITE (5, 303) LABEL, LINE
303 FORMAT (1H ,I4,111A1)
LAB10 = LAB10 +10
GO TO 12
10 WRITE (5, 304) LINE
304 FORMAT (1H ,4X,111A1)
12 LINE(J) = BLANK
RETURN
END

```

```

BACKG4DS
BACKG5DS
BACKG6DS
BACKG7DS
BACKG8DS
BACKG9DS
BACKH0DS
BACKH1DS
BACKH2DS
BACKH3DS
BACKH4DS
BACKH5DS
BACKH6DS
BACKH7DS
BACKH8DS
BACKH9DS
BACKJ0DS
BACKJ1DS
BACKJ2DS
BACKJ3DS
BACKJ4DS
BACKJ5DS
BACKJ6DS
BACKJ7DS
BACKJ8DS
BACKJ9DS
BACKK0DS
BACKK1DS
BACKK2DS
BACKK3DS

```

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